

Mr. C. Lance Terry
TU Electric
Group Vice President, Nuclear
Attn: Regulatory Affairs Department
P. O. Box 1002
Glen Rose, TX 76043

September 30, 1996

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - AMENDMENT
NOS. 52 AND 38 TO FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89
(TAC NOS. M96244 AND M96245)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment Nos. 52 and 38 to Facility Operating License Nos. NPF-87 and NPF-89 for the Comanche Peak Steam Electric Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated July 31, 1996 (TXX-96433).

These amendments revise core safety limit curves (TS Figure 2.1-1a) and new N-16 setpoint values and parameters (TS Table 2.1-1) for Unit 1, and reference to topical report RXE-95-001-P as an approved methodology for small break loss of coolant accident (LOCA) analysis for Units 1 and 2.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 52 to NPF-87
2. Amendment No. 38 to NPF-89
3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 30, 1996

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Sincerely,

A handwritten signature in cursive script, reading "Timothy J. Polich", is written over the typed name.

Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 52 to NPF-87
2. Amendment No. 38 to NPF-89
3. Safety Evaluation

cc w/encls: See next page

Mr. C. Lance Terry
TU Electric Company

Comanche Peak, Units 1 and 2

cc:

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Regional Administrator, Region IV
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Glen Rose, TX 76043

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Environmental Policy
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Texas Department of Health
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-445
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. NPF-87

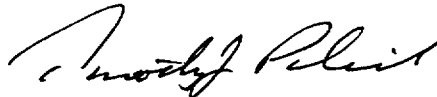
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated July 31, 1996 (TX-96433), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 52, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: September 30, 1996



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 2
DOCKET NO. 50-446
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. NPF-89

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated July 31, 1996 (TXX-96433), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-89 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 38, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. TU Electric shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: September 30, 1996

ATTACHMENT TO LICENSE AMENDMENT NOS. 52 AND 38

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

2-2
2-9
2-10
2-11
6-11

INSERT

2-2
2-9
2-10
2-11
6-11

2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS

REACTOR CORE

2.1.1 The combination of THERMAL POWER, pressurizer pressure, and the highest operating loop coolant temperature (T_{avg}) shall not exceed the limits shown in Figure 2.1-1.

APPLICABILITY: MODES 1 and 2.

ACTION:

Whenever the point defined by the combination of the highest operating loop average temperature and THERMAL POWER has exceeded the appropriate pressurizer pressure line, be in HOT STANDBY within 1 hour, and comply with the requirements of Specification 6.7.1.

REACTOR COOLANT SYSTEM PRESSURE

2.1.2 The Reactor Coolant System pressure shall not exceed 2735 psig.

APPLICABILITY: MODES 1, 2, 3, 4, and 5.

ACTION:

MODES 1 and 2:

Whenever the Reactor Coolant System pressure has exceeded 2735 psig, be in HOT STANDBY with the Reactor Coolant System pressure within its limit within 1 hour, and comply with the requirements of Specification 6.7.1.

MODES 3, 4 and 5:

Whenever the Reactor Coolant System pressure has exceeded 2735 psig, reduce the Reactor Coolant System pressure to within its limit within 5 minutes, and comply with the requirements of Specification 6.7.1.

Unit 1 Reactor Core Safety Limits

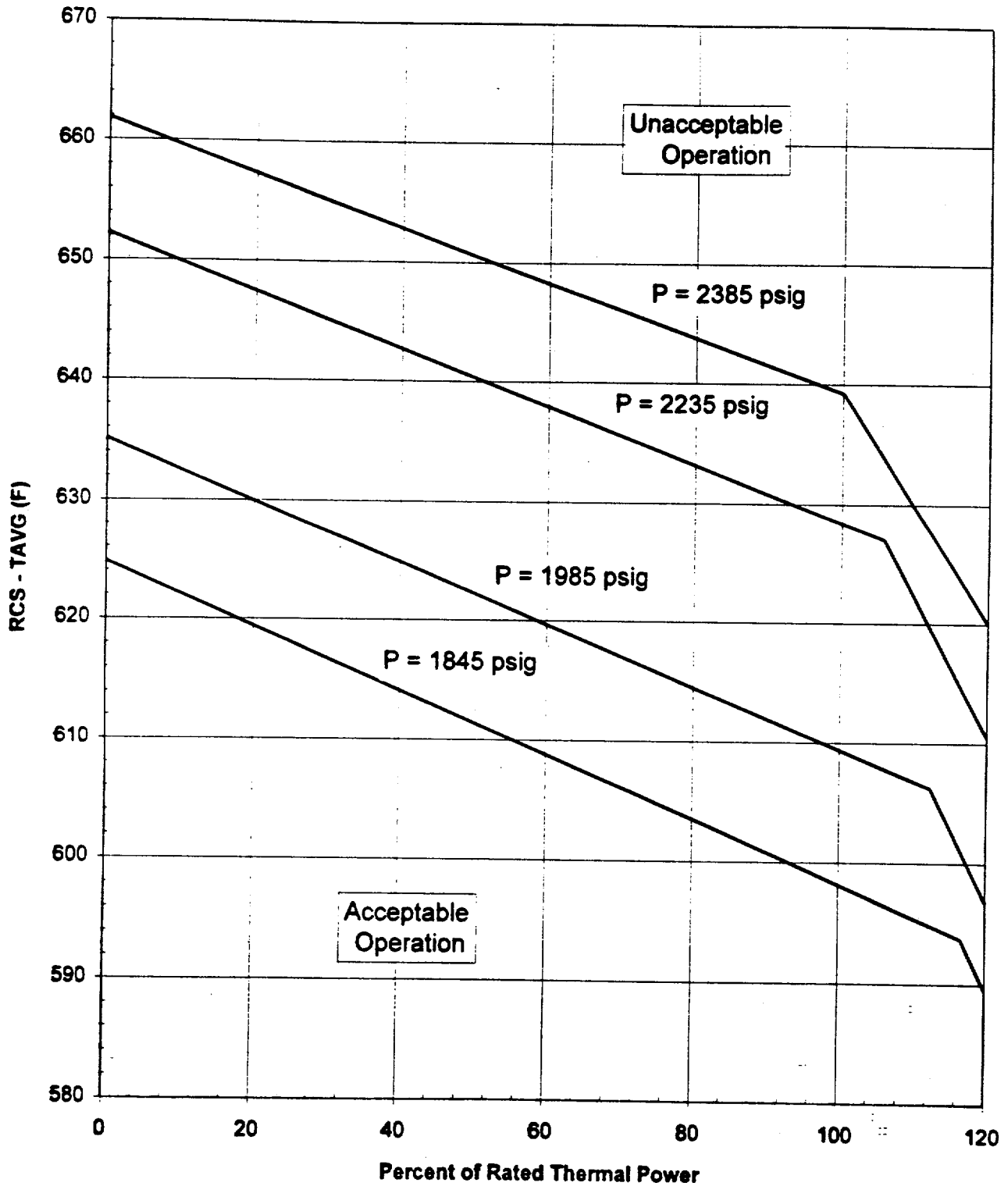


FIGURE 2.1-1a

UNIT 1 REACTOR CORE SAFETY LIMITS

TABLE 2.2-1 (Continued)
TABLE NOTATIONS

NOTE 1: Overtemperature N-16

$$N = K_1 - K_2 \left[\frac{1 + \tau_1 s}{1 + \tau_2 s} T_c - T_c^o \right] + K_3 (P - P^1) - f_1 (\Delta q)$$

Where:	N	= Measured N-16 Power by ion chambers,
	T_c	= Cold leg temperature, °F,
	T_c^o	= 560.5°F for Unit 1, 560.8°F for Unit 2 - Reference T_c at RATED THERMAL POWER,
	K_1	= 1.150,
	K_2	= 0.0173/°F for Unit 1 0.0138/°F for Unit 2
	$\frac{1 + \tau_1 s}{1 + \tau_2 s}$	= The function generated by the lead-lag controller for T_c dynamic compensation,
	τ_1, τ_2	= Time constants utilized in the lead-lag controller for T_c , $\tau_1 \geq 10$ s, and $\tau_2 \leq 3$ s,
	K_3	= 0.000890/psig for Unit 1 0.000720/psig for Unit 2

TABLE 2.2-1 (Continued)

TABLE NOTATIONS (Continued)

NOTE 1: (Continued)

- P = Pressurizer pressure, psig,
P' ≥ 2235 psig (Nominal RCS operating pressure),
S = Laplace transform operator, s^{-1} ,

and $f_1(\Delta q)$ is a function of the indicated difference between top and bottom halves of detectors of the power-range neutron ion chambers; with gains to be selected based on measured instrument response during plant STARTUP tests such that:

For Unit 1

- (i) for $q_t - q_b$ between -65% and +4.6%, $f_1(\Delta q) = 0$, where q_t and q_b are percent RATED THERMAL POWER in the top and bottom halves of the core respectively, and $q_t + q_b$ is total THERMAL POWER in percent of RATED THERMAL POWER,
- (ii) for each percent that the magnitude of $q_t - q_b$ exceeds -65%, the N-16 Trip Setpoint shall be automatically reduced by 0.0%* of its value at RATED THERMAL POWER, and
- (iii) for each percent that the magnitude of $q_t - q_b$ exceeds +4.6%, the N-16 Trip Setpoint shall be automatically reduced by 3.04% of its value at RATED THERMAL POWER.

*No setpoint reduction is required for the span of the ΔI indication.

TABLE 2.2-1 (Continued)
TABLE NOTATIONS (Continued)

NOTE 1: (Continued)

For Unit 2

- (i) for $q_t - q_b$ between -65% and +2.5%, $f_1(\Delta q) = 0$, where q_t and q_b are percent RATED THERMAL POWER in the top and bottom halves of the core respectively, and $q_t + q_b$ is total THERMAL POWER in percent of RATED THERMAL POWER,
- (ii) for each percent that the magnitude of $q_t - q_b$ exceeds -65%, the N-16 Trip Setpoint shall be automatically reduced by 1.86% of its value at RATED THERMAL POWER, and
- (iii) for each percent that the magnitude of $q_t - q_b$ exceeds +2.5%, the N-16 Trip Setpoint shall be automatically reduced by 1.65% of its value at RATED THERMAL POWER.

NOTE 2: The channel's maximum Trip Setpoint shall not exceed its computed Trip Setpoint by more than 3.64% of span for Unit 1 or 1.88% of span for Unit 2.

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

Reference 18) is for Unit 2 only:

- 18). WCAP-9220-P-A, Rev. 1, "WESTINGHOUSE ECCS EVALUATION MODEL- 1981 Version", February 1982 (W Proprietary).
- 19). RXE-94-001-A, "Safety Analysis of Postulated Inadvertent Boron Dilution Event in Modes 3, 4, and 5," February 1994. (Methodology for Specifications 3/4.1.1.1, 3/4.1.1.2, 3/4.1.2.2, 3/4.1.2.4, and 3/4.1.2.6 - Shutdown Margin.)
- 20.) RXE-95-001-P, "Small Break Loss of Coolant Accident Analysis Methodology," December 1995.

6.9.1.6c The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as SHUTDOWN MARGIN, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6d The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, special reports shall be submitted to the Regional Administrator of the Regional Office of the NRC within the time period specified for each report.

6.10 NOT USED

6.11 NOT USED

6.12 HIGH RADIATION AREA

6.12.1 Pursuant to paragraph 10 CFR 20.1601(c), in lieu of the "control device" or "alarm signal" required by paragraph 10 CFR 20.1601(a), each high radiation area, as defined in 10 CFR 20, in which the intensity of radiation is equal to or less than 1000 mrem/h at 30 cm (12 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Radiation Protection Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with dose rates equal to or less than 1000 mrem/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

COMANCHE PEAK - UNITS 1 AND 2

6-11 Unit 1 - Amendment No. ~~21, 42, 44, 50, 52~~
Unit 2 - Amendment No. ~~7, 28, 30, 36, 38~~

HIGH RADIATION AREA (Continued)

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area; or
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and individuals have been made knowledgeable of them; or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to individuals with radiation levels greater than 1000 mrem/h at 30 cm (12 in.) but less than 500 rads in one hour at one meter from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Manager on duty and/or radiation protection supervision. Doors shall remain locked except during periods of access by individuals under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed circuit TV cameras) continuous surveillance may be made by individuals qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

For isolated high radiation areas accessible to individuals with radiation levels of greater than 1000 mrem/h at 30 cm but less than 500 rads in one hour at one meter that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the isolated area, that isolated area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

6.13 NOT USED**6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)**

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained for the duration of the unit Operating License. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 52 AND 38 TO
FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89
TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2
DOCKET NOS. 50-445 AND 50-446

1.0 INTRODUCTION

By application dated July 31, 1996 (TXX-96433), Texas Utilities Electric Company (TU Electric/the licensee) requested changes to the Technical Specifications (TSs) (Appendix A to Facility Operating License Nos. NPF-87 and NPF-89) for the Comanche Peak Steam Electric Station (CPSES), Units 1 and 2. The proposed changes reflect revised core safety limit curves (TS Figure 2.1-1a) and new N-16 setpoint values and parameters (TS Table 2.1-1) for Unit 1, and reference to topical report RXE-95-001-P as an approved methodology for small break loss of coolant accident (LOCA) analysis for Units 1 and 2.

Specifically, the following terms would be changed in Note 1 to TS Table 2.2-1 for Unit 1:

- a) K_2 from 0.0134/°F to 0.0173/°F,
- b) K_3 from 0.000719/psig to 0.000890/psig,
- c) q_t-q_b range from [-65% and +4%] to [-65% and +4.6%],
- d) Overtemperature N-16 setpoint reduction from 1.81% to 0.0% for each percent that the magnitude of q_t-q_b exceeds -65%,
- e) Overtemperature N-16 setpoint reduction from 2.26% to 3.04% for each percent that the magnitude of q_t-q_b exceeds +4.6%,
- f) Footnote identifying that no setpoint reduction is required for the span of the ΔI indication.

In Note 2 to TS Table 2.2-1, the maximum amount by which the N-16 trip setpoint is allowed to exceed the computed trip setpoint would be increased from 3.51% to 3.64% for Unit 1.

In addition, TS 6.9.1.6b would be revised to allow the use of the methodology presented in the TU Electric topical report, "Small Break Loss of Coolant Accident Analysis Methodology," RXE-95-001-P, to analyze the effects of the small break LOCA for CPSES, Units 1 and 2. This topical report was reviewed and approved by the NRC staff in a safety evaluation dated September 4, 1996.

2.0 EVALUATION

Based on analyses of the core configuration for CPSES, Unit 1, Cycle 6, TU Electric has determined that revised core safety limit curves and overtemperature N-16 reactor trip setpoints are required. The core safety limits are the loci of points of thermal power, reactor coolant system (RCS) pressure and average temperature below which either the calculated departure from nucleate boiling ratio (DNBR) is no less than the safety limit value, or the average enthalpy at the vessel exit is less than the enthalpy of saturated liquid. The overtemperature N-16 reactor trip setpoint is calculated such that a reactor trip will be initiated before the core safety limits are exceeded.

The transient most affected by the change in the overtemperature N-16 trip setpoint is the control rod withdrawal event from full power conditions. TU Electric reanalyzed this event using NRC-approved methodologies, as specified in TS 6.9.1.6b, and the revised overtemperature N-16 trip setpoint. The event was shown to terminate before a departure from nucleate boiling (DNB) condition was reached. Since this limiting event, as well as the analyses of the revised core safety limits and the overtemperature N-16 trip setpoint, was performed in accordance with the NRC-approved methodologies listed in TS 6.9.1.6b, and satisfied all relevant acceptance criteria, the proposed TS changes to the core safety limits and the overtemperature N-16 trip are acceptable.

TS 6.9.1.6b would be revised to allow the use of the methodology presented in the TU Electric topical report, RXE-95-001-P, "Small Break Loss of Coolant Accident Analysis Methodology," to analyze the effects of the small break LOCA for both Units of CPSES. This topical report was reviewed and approved by the NRC staff in a safety evaluation dated September 4, 1996. Use of the NRC approved methodology will ensure that values for cycle-specific parameters will be determined such that applicable operational transient and accident limits of the safety analysis are met. Therefore, the proposed change is acceptable.

3.0 CONCLUSION

The staff has reviewed the proposed changes to the CPSES, Units 1 and 2 TS to reflect revised core safety limit curves (TS Figure 2.1-1a), new N-16 setpoint values and parameters (TS Table 2.1-1), and the inclusion of RXE-95-001-P as an approved methodology for small break LOCA analysis. Based on the NRC evaluation given above, the proposed changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 44362). The amendment also changes reporting and recordkeeping requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Kopp

Date: September 30, 1996