

October 5, 1994

DISTRIBUTION:

Mr. C. Lance Terry
Group Vice President, Nuclear
TU Electric
400 North Olive Street, L.B. 81
Dallas, Texas 75201

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SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - AMENDMENT
NOS. 28 AND 14 TO FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89
(TAC NOS. M89190 AND M89191)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment Nos. 28 and 14 to Facility Operating License Nos. NPF-87 and NPF-89 for the Comanche Peak Steam Electric Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 28, 1994.

The amendments revise the TS by deleting reference to a methodology that is no longer applicable, WCAP-9220-P-A, "WESTINGHOUSE ECCS EVALUATION MODEL," and adds a reference CPSES Topical Report RXE-91-005, "Methodology for Reactor Core Response to Steamline Break Events." The staff has previously reviewed Topical Report RXE-91-005 and found the methodology acceptable for use by TU Electric for Comanche Peak Steam Electric Station, Units 1 and 2, subject to the constraints of the NRC safety evaluation related to topical report RXE-91-005, dated December 30, 1993.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by:

Thomas A. Bergman, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

9410140201 941005
PDR ADOCK 05000445
P PDR

Docket Nos. 50-445
and 50-446

Enclosures:

1. Amendment No. 28 to NPF-87
2. Amendment No. 14 to NPF-89
3. Safety Evaluation

cc w/encls:

See next page

*See previous concurrence

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DATE	10/4/94	8/4/94	/ /94	9/20/94	10/4/94

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 5, 1994

Mr. C. Lance Terry
Group Vice President, Nuclear
TU Electric
400 North Olive Street, L.B. 81
Dallas, Texas 75201

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - AMENDMENT
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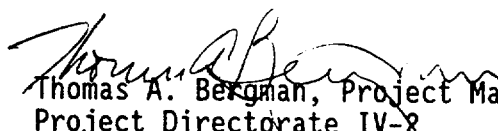
Dear Mr. Terry:

The Commission has issued the enclosed Amendment Nos. 28 and 14 to Facility Operating License Nos. NPF-87 and NPF-89 for the Comanche Peak Steam Electric Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 28, 1994.

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A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,


Thomas A. Bergman, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-445
and 50-446

Enclosures:

1. Amendment No.28 to NPF-87
2. Amendment No.14 to NPF-89
3. Safety Evaluation

cc w/encs:
See next page

Mr. C. Lance Terry
TU Electric Company

CC:
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U.S. Nuclear Regulatory Commission
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Granbury, Texas 76048

Regional Administrator, Region IV
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Comanche Peak, Units 1 and 2

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Texas Department of Health
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Austin, Texas 78756

Honorable Dale McPherson
County Judge
P. O. Box 851
Glen Rose, Texas 76043

Office of the Governor
ATTN: Susan Rieff, Director
Environmental Policy
P. O. Box 12428
Austin, Texas 78711



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-445
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28
License No. NPF-87

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated March 28, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

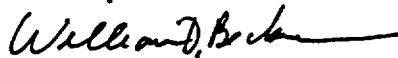
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2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 28, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 5, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 2
DOCKET NO. 50-446
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14
License No. NPF-89

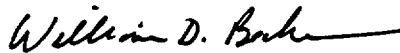
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated March 28, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-89 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 14, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. TU Electric shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 5, 1994

ATTACHMENT TO LICENSE AMENDMENT NOS. 28 AND 14

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a marginal line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

6-21

INSERT

6-21

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

- 5). WCAP-10216-P-A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL F_0 SURVEILLANCE TECHNICAL SPECIFICATION," June 1983 (W Proprietary).
(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor ($W(z)$) surveillance requirements for F_0 Methodology).)
- 6). WCAP-10079-P-A, "NOTRUMP, A NODAL TRANSIENT SMALL BREAK AND GENERAL NETWORK CODE," August 1985, (W Proprietary).
- 7). WCAP-10054-P-A, "WESTINGHOUSE SMALL BREAK ECCS EVALUATION MODEL USING THE NOTRUMP CODE", August 1985, W Proprietary).
- 8). WCAP-11145-P-A, "WESTINGHOUSE SMALL BREAK LOCA ECCS EVALUATION MODEL GENERIC STUDY WITH THE NOTRUMP CODE", October 1986, W Proprietary).
- 9). RXE-90-006-P, "Power Distribution Control Analysis and Overtemperature N-16 and Overpower N-16 Trip Setpoint Methodology," February 1991.
(Methodology for Specification 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor.)
- 10). RXE-88-102-P, "TUE-1 Departure from Nucleate Boiling Correlation", January 1989.
- 11). RXE-88-102-P, Sup. 1, "TUE-1 DNB Correlation - Supplement 1", December 1990.
- 12). RXE-89-002, "VIPRE-01 Core Thermal-Hydraulic Analysis Methods for Comanche Peak Steam Electric Station Licensing Applications", June 1989.
- 13). RXE-91-001, "Transient Analysis Methods for Comanche Peak Steam Electric Station Licensing Applications", February 1991.
- 14). RXE-91-002, "Reactivity Anomaly Events Methodology", May 1991.
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limit, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
- 15). RXE-90-007, "Large Break Loss of Coolant Accident Analysis Methodology", December 1990.
- 16). TXX-88306, "Steam Generator Tube Rupture Analysis", March 15, 1988.
- 17). RXE-91-005, "Methodology for Reactor Core Response to Steamline Break Events," May, 1991.

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

Reference 18) is for Unit 2 only:

- 18). WCAP-9220-P-A, Rev. 1, "WESTINGHOUSE ECCS EVALUATION MODEL- 1981 Version", February 1982 (W Proprietary).

6.9.1.6c The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as SHUTDOWN MARGIN, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6d The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 28 AND 14 TO
FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89
TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2
DOCKET NOS. 50-445 AND 50-446

1.0 INTRODUCTION

By application dated March 28, 1994, Texas Utilities Electric Company (TU Electric/the licensee) requested changes to the Technical Specifications (TSs) (Appendix A to Facility Operating License Nos. NPF-87 and NPF-89) for the Comanche Peak Steam Electric Station, Units 1 and 2. The proposed request includes two changes to TSs Section 6.9.1.6b, "Core Operating Limits Report." The first proposed change is to delete topical report WCAP-9220-P-A, "Westinghouse ECCS Evaluation Model." The second proposed change is to add a topical report for steamline break analysis, RXE-91-005, "Methodology for Reactor Core Response to Steamline Break Events."

2.0 BACKGROUND

TS Section 6.9.1.6b lists references that contain analytical methods approved by the NRC for the determination of core operating limits. TU Electric has developed analytical methods for determining the core operating limits. Prior to the fourth fuel cycle, the Westinghouse ECCS evaluation report (WCAP-9220-P-A) had been used to analyze large break loss-of-coolant accidents (LOCAs) for Unit 1. Beginning with the fourth fuel cycle for Unit 1, large break LOCA analyses are being performed using the TU Electric methodology RXE-90-007, "Large Break Loss of Coolant Accident Analysis Methodology." This methodology was approved for use by the licensee for CPSES Units 1 and 2 in a staff safety evaluation dated November 16, 1993, in support of License Amendments 21 and 7 to the Units 1 and 2 licenses, respectively. In a staff safety evaluation dated December 30, 1993, Topical Report RXE-91-005, "Methodology for Reactor Core Response to Steamline Break Events," was approved for use by the licensee for CPSES Units 1 and 2, subject to the conditions stated in that safety evaluation.

3.0 EVALUATION

The first proposed change would delete a methodology that is no longer applicable at CPSES. As noted in the staff's November 16, 1993, safety evaluation, WCAP-9220-P-A was only applicable through fuel cycle 3.

Effective with fuel cycle 4 operations, the licensee was to use RXE-90-007, "Large Break Loss of Coolant Accident Analysis Methodology," in place of WCAP-9220-P-A. The staff determined, in the November 16, 1993, safety evaluation, that RXE-90-007, in conjunction with other approved methodologies, would ensure that all applicable limits of the safety analyses are met for reload core configurations. This conclusion remains valid, and therefore, WCAP-9220-P-A may be deleted as it is no longer applicable.

Topical Report RXE-91-005 was approved for use for CPSES, Units 1 and 2, subject to the conditions stated in the staff safety evaluation dated December 30, 1993. In that safety evaluation, the staff concluded that the use of RXE-91-005, under the conditions stated in the safety evaluation, was acceptable because it would result in predictions with adequate assurances of conservatism. This conclusion remains valid. The inclusion of this methodology into TS is acceptable as it will ensure, in conjunction with other approved methodologies, that all applicable limits of the safety analyses are met for reload core configurations.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Bryant

Date: October 5, 1994