

March 6, 1992

Docket No. 50-445

Mr. William J. Cahill, Jr.
Executive Vice President
TU Electric
400 North Olive Street, L.B. 81
Dallas, Texas 75201

DISTRIBUTION:

Docket File	BBoger
NRC/Local PDR	GHill (4)
SFlanders	Wanda Jones
PDIV-2 Reading	JClifford
EPeyton	ACRS (10)
TBergman (2)	GPA/PA
MVirgilio	OC/LFMB
OGC	DHagan*
LYandell, RIV	Plant File
JWermiel	SFlanders

Dear Mr. Cahill:

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1 - AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE NO. NPF-87 (TAC NO. M80265)

The Commission has issued the enclosed Amendment No. 9 to Facility Operating License No. NPF-87 for the Comanche Peak Steam Electric Station, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application dated April 22, 1991, as supplemented by letter dated November 4, 1991.

The amendment revises Sections 6.2 and 6.5 of the Comanche Peak Steam Electric Station Unit 1 Technical Specifications to reflect organizational and title changes at TU Electric.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

Thomas A. Bergman, Acting Project Manager
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 9 to NPF-87
2. Safety Evaluation

cc w/enclosures:
See next page

OFC	: PDIV-2/LA	: PDIV-2/PE	: PDIV-2/(A)PM:	OGC	: PDIV-2/D	: LHE/NRR	:
NAME	: EPeyton	: SFlanders	: TBergman	: JWermiel	: SBlack	: JWermiel	:
DATE	: 1/9/92	: 2/11/92	: 2/11/92	: 2/11/92	: 3/2/92	: 2/12/92	:

OFFICIAL RECORD COPY
Document Name: CP AMENDMENT/TAC M80265

9203180506 920306
PDR ADOCK 05000445
P PDR

JW

March 6, 1992

cc w/enclosures:

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P. O. Box 1029
Granbury, Texas 76048

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Mrs. Juanita Ellis, President
Citizens Association for Sound Energy
1426 South Polk
Dallas, Texas 75224

Owen L. Thero, President
Quality Technology Company
Lakeview Mobile Home Park, Lot 35
4793 East Loop 820 South
Fort Worth, Texas 76119

Mr. Roger D. Walker
Manager, Nuclear Licensing
Texas Utilities Electric Company
400 North Olive Street, L.B. 81
Dallas, Texas 75201

Texas Utilities Electric Company
c/o Bethesda Licensing
3 Metro Center, Suite 610
Bethesda, Maryland 20814

William A. Burchette, Esq.
Counsel for Tex-La Electric
Cooperative of Texas
Jorden, Schulte, & Burchette
1025 Thomas Jefferson Street, N.W.
Washington, D.C. 20007

GDS Associates, Inc.
Suite 720
1850 Parkway Place
Marietta, Georgia 30067-8237

Jack R. Newman, Esq.
Newman & Holtzinger
1615 L Street, N.W.
Suite 1000
Washington, D.C. 20036

Chief, Texas Bureau of Radiation Control
Texas Department of Health
1100 West 49th Street
Austin, Texas 78756

Honorable Dale McPherson
County Judge
P.O. Box 851
Glen Rose, Texas 76043



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TEXAS UTILITIES ELECTRIC COMPANY, ET AL.*
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-445
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. NPF-87

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric) acting for itself and as agent for Texas Municipal Power Agency (licensees) dated April 22, 1991, as supplemented by letter dated November 4, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*The current owners of the Comanche Peak Steam Electric Station are: Texas Utilities Electric Company and Texas Municipal Power Agency. Transfer of ownership from Texas Municipal Power Agency to Texas Utilities Electric Company was previously authorized by Amendment No. 9 to Construction Permit CPPR-126 on August 25, 1988 to take place in 10 installments as set forth in the Agreement attached to the application for Amendment dated March 4, 1988. At the completion thereof, Texas Municipal Power Agency will no longer retain any ownership interest.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 7 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Suzanne C. Black

Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 6, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 9

FACILITY OPERATING LICENSE NO. NPF-87

DOCKET NO. 50-445

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

6-1
6-4
6-7
6-8
6-9
6-11

INSERT

6-1
6-4
6-7
6-8
6-9
6-11

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President, Nuclear Operations shall be responsible for overall operation of the site, while the Plant Manager shall be responsible for operation of the unit. The Vice President, Nuclear Operations and the Plant Manager shall each delegate in writing the succession to this responsibility during their absence.

6.1.2 The Shift Supervisor (or during his absence from the control room, a designated individual, see Table 6.2-1) shall be responsible for the control room command function. A management directive to this effect, signed by the Vice President, Nuclear Operations shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 ONSITE AND OFFSITE ORGANIZATION

An onsite and an offsite organization shall be established for unit operation and corporate management, respectively. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organizational charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in the equivalent forms of documentation. These requirements shall be documented in the FSAR.
- b. The Vice President, Nuclear Operations shall be responsible for overall site safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Group Vice President, Nuclear Engineering and Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out the radiation protection and quality assurance functions may report to the appropriate manager onsite; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 UNIT STAFF

The unit organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

ADMINISTRATIVE CONTROLS

UNIT STAFF (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;
- c. A Radiation Protection Technician* and a Chemistry Technician* shall be on site when fuel is in the reactor;
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least five members* shall be maintained on site at all times. The Fire Brigade shall not include the Shift Supervisor and the two other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency;
- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, Radiation Protection Technicians, auxiliary operators, and key maintenance personnel).

The amount of overtime worked by unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12); and
- g. The Shift Operations Manager shall hold a Senior Reactor Operator license.

*The Radiation Protection and the Chemistry Technicians and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION
SINGLE UNIT FACILITY

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3, or 4	MODE 5 or 6
SS	1	1
SRO	1	None
RO	2	1
AO	2	1
STA	1*	None

- SS - Shift Supervisor with a Senior Operator license on Unit 1
- SRO - Individual with a Senior Operator license on Unit 1
- RO - Individual with an Operator license on Unit 1
- AO - Auxiliary Operator
- STA - Shift Technical Advisor

The shift crew composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the control room while the unit is in MODE 1, 2, 3, or 4, an individual with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Shift Supervisor from the control room while the unit is in MODE 5 or 6, an individual with a valid Senior Operator license or Operator license shall be designated to assume the control room command function.

*The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Shift Supervisor or the individual with a Senior Operator license meets the qualifications described in Option 1 of the Commission Policy Statement on Engineering Expertise (50 FR 43621, October 28, 1985).

ADMINISTRATIVE CONTROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of unit design and operating experience information, including units of similar design, which may indicate areas for improving unit safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving unit safety to the Group Vice President, Nuclear Engineering and Operations.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers located on site. Each shall have a bachelor's degree in engineering or related science and at least 3 years professional level experience in his field.

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of unit activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

RECORDS

6.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to the Group Vice President, Nuclear Engineering and Operations.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor (STA) shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline and shall have received specific training in the response and analysis of the unit for transients and accidents, and in unit design and layout, including the capabilities of instrumentation and controls in the control room.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI-N18.1-1971 for comparable positions, except for the Radiation Protection Manager** who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, for a Radiation Protection Manager. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees. (Prior to meeting

*Not responsible for sign-off function.

**Until the Radiation Protection Manager meets all qualifications per Regulatory Guide 1.8, September 1975, the Assistant Radiation Protection Manager, who meets all those qualifications, shall be responsible for the technical aspects of the program.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- i. Review of the Security Plan and shall submit recommended changes to the ORC;
- j. Review of the Emergency Plan and shall submit recommended changes to the ORC;
- k. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems;
- l. Review of any accidental, unplanned or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President, Nuclear Operations, and to the ORC;
- m. Review of Unit operations to detect potential hazards to nuclear safety;
- n. Investigations or analysis of special subjects as requested by the Chairman of the ORC or the Vice President, Nuclear Operations;
- o. Review of the Fire Protection Report and implementing procedures and submittal of recommended changes to the ORC; and
- p. Review of the Technical Requirements Manual and revision thereto.

6.5.1.7 The SORC shall:

- a. Recommend in writing to the designated manager (see Specification 6.5.3) approval or disapproval of items considered under Specification 6.5.1.6a through e prior to their implementation;
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6a. through e. constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Group Vice President, Nuclear Engineering and Operations and the Operations Review Committee of disagreement between the SORC and the designated manager (see Specification 6.5.3); however, the Vice President, Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.1.8 The SORC shall maintain written minutes of each SORC meeting that, at a minimum, document the results of all SORC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President, Nuclear Operations and the Operations Review Committee.

6.5.2 OPERATIONS REVIEW COMMITTEE (ORC)

FUNCTION

6.5.2.1 The ORC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The ORC shall report to and advise the Group Vice President, Nuclear Engineering and Operations on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

COMPOSITION

6.5.2.2 The ORC shall be composed of at least six individuals of whom no more than a minority are members having line responsibility for operations at CPSES. The Chairman and all members will be appointed by the Group Vice President, Nuclear Engineering and Operations.

The ORC Chairman shall hold a Bachelor's degree in an engineering or physical science field or equivalent experience and a minimum of 6 years technical managerial experience.

The ORC members shall hold a Bachelor's degree in an engineering or physical science field or equivalent experience and a minimum of 5 years technical experience with the exception of CASE's representative, Mrs. Juanita Ellis, and CASE's alternate designated by Mrs. Ellis. It is the responsibility of the Chairman to ensure experience and competence is available to review problems in areas listed in Specification 6.5.2.1a. through h. To a large measure, this experience and competence rests with the membership of the ORC. In specialized areas, this experience may be provided by personnel who act as consultants to the ORC.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.2.3 The alternate for the Chairman and all alternate members shall be appointed in writing by the Group Vice President, Nuclear Engineering and Operations to serve on a temporary basis; however, no more than two alternates shall participate as voting members in ORC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the Chairman, ORC to provide expert advice to the ORC.

MEETING FREQUENCY

6.5.2.5 The ORC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.2.6 The quorum of the ORC necessary for the performance of the ORC review and audit functions of these Technical Specifications shall consist of not less than a majority of the appointed individuals (or their alternates) and the Chairman or his designated alternate. No more than a minority of the quorum shall have line responsibility for operation of the unit.

CASE's representative, Mrs. Juanita Ellis or CASE's alternate designated by Mrs. Ellis, during the period she serves as a member of the ORC, shall be entitled to all of the rights and privileges that all other individuals have as members of the ORC, but for the purposes of this paragraph shall not be counted for establishing or meeting quorum requirements.

REVIEW

6.5.2.7 The ORC shall be responsible for the review of:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the SORC.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the ORC. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training, and qualifications of the entire unit staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR 50, at least once per 24 months;
- e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel;
- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- h. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months;
- j. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months;
- k. Any other area of unit operation considered appropriate by the ORC or the Group Vice President, Nuclear Engineering and Operations; and
- l. The performance of activities required by the Technical Requirements Manual at least once per 24 months.

RECORDS

6.5.2.9 Records of ORC activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each ORC meeting shall be prepared, approved, and forwarded to the Vice President, Nuclear Operations and Group Vice President, Nuclear Engineering and Operations within 14 days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.2.7 shall be prepared, approved, and forwarded to the Vice President, Nuclear Operations and Group Vice President, Nuclear Engineering and Operations within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.2.8 shall be forwarded to the Vice President, Nuclear Operations and Group Vice President, Nuclear Engineering and Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.5.3 TECHNICAL REVIEW AND CONTROLS

6.5.3.1 Activities which affect nuclear safety shall be conducted as follows:

- a. Procedures required by Specification 6.8 and other procedures which affect plant nuclear safety, and changes thereto, shall be prepared, reviewed and approved. Each such procedure or procedure change shall be reviewed by a qualified individual/group other than the individual/group which prepared the procedure or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. The Vice President, Nuclear Operations, shall approve Station Administrative Procedures, Security Plan Implementing Procedures, and Emergency Plan Implementing Procedures. Other procedures shall be approved by the appropriate

ADMINISTRATIVE CONTROLS

TECHNICAL REVIEW AND CONTROLS (Continued)

department manager as previously designated by the Vice President, Nuclear Operations, in writing. Individuals responsible for procedure reviews shall be members of the Nuclear Operations Management Staff previously designated by the Vice President, Nuclear Operations. Changes to procedures which do not change the intent of approved procedures may be approved for implementation by two members of the Nuclear Operations Management Staff, at least one of whom holds a Senior Operator License, provided such approval is prior to implementation and is documented. Such changes shall be approved by the original approval authority within 14 days of implementation;

- b. Proposed tests and experiments which affect plant nuclear safety shall be prepared, reviewed, and approved. Each such test or experiment shall be reviewed by a qualified individual/group other than the individual/group which prepared the proposed test or experiment. Proposed test and experiments shall be approved before implementation by the Plant Manager. Individuals responsible for conducting such reviews shall be members of the Nuclear Operations Management Staff previously designated by the Vice President, Nuclear Operations;
- c. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Chief Engineer. Each such modification shall be reviewed by a qualified individual/group meeting the experience requirements of ANSI N18.1-1971, Section 4.6 other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Individuals/groups responsible for conducting such reviews shall be previously designated by the Chief Engineer. Proposed modifications to plant nuclear safety-related structures, systems and components shall be approved by the Plant Manager prior to implementation;
- d. Individuals responsible for reviews performed in accordance with the requirements of Specifications 6.5.3.1a and 6.5.3.1b, shall be members of the Nuclear Operations Management staff previously designated by the Vice President, Nuclear Operations. Each such review shall include a determination of whether or not additional cross-disciplinary review is necessary. If deemed necessary, such review shall be done in accordance with the appropriate qualification requirements;
- e. Each review shall include a determination of whether or not an unreviewed safety question is involved. For items involving unreviewed safety questions, NRC approval shall be obtained prior to the Plant Manager, approval for implementation; and
- f. The Security Plan and Emergency Plan, and implementing procedures, shall be reviewed at least once per 12 months. Recommended changes to the implementing procedures shall be approved by the Vice President,



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE NO. NPF-87

TEXAS UTILITIES ELECTRIC COMPANY, ET AL.

COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1

DOCKET NO. 50-445

1.0 INTRODUCTION

By application dated April 22, 1991, and supplemented by letter dated November 4, 1991, Texas Utilities Electric Company (TU Electric, the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-87) for the Comanche Peak Steam Electric Station (CPSES), Unit No. 1. The proposed changes would revise Sections 6.2 and 6.5 of the Technical Specification (TS) to reflect organizational and title changes at TU Electric. Specifically, the licensee has requested changes in the reporting responsibilities for the CPSES Independent Safety Engineering Group (ISEG) and the title of the Executive Vice President, Nuclear Engineering and Operations.

2.0 EVALUATION

By application dated April 22, 1991 TU Electric proposed changes to CPSES technical specification paragraphs 6.2.3.1 and 6.2.3.4 to redefine the reporting responsibilities of the ISEG. TS paragraphs 6.2.3.1 and 6.2.3.4 require the ISEG recommendations and reports to be provided to the Vice President, Nuclear Engineering, who then reports to the Executive Vice President, Nuclear Engineering and Operations. TU Electric has requested to change paragraphs 6.2.3.1 and 6.2.3.4 to indicate that the ISEG reports directly to the Executive Vice President, Nuclear Engineering and Operations. In the supplemental application dated November 4, 1991, TU Electric proposed to change the title of the Executive Vice President, Nuclear Engineering and Operations to Group Vice President, Nuclear Engineering and Operations.

The changes described above do not represent any changes that would adversely affect or endanger the health or safety of the general public or involve a significant safety hazard. The changes are administrative in nature and do not change plant operations or any commitments made by the licensee. Therefore, the staff finds the proposed changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Flanders

Date: March 6, 1992