

Mr. Charles M. Dugger
Vice President Operations
Entergy Operations, Inc.
P. O. Box B
Killona, LA 70066

November 20, 1997

SUBJECT: ISSUANCE OF AMENDMENT NO. 136 TO FACILITY OPERATING LICENSE
NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3 (TAC NO. M95370)

Dear Mr. Dugger:

The Commission has issued the enclosed Amendment No.136 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 6, 1996.

The amendment changes the Appendix A TSs by correcting several inconsistencies. These changes effect Table 3.3-4, the allowable values of the parameters; Table 4.3-2 Item 5.c, Safety Injection System Automatic Actuation Logic, modes for which the surveillances are required; and TS 4.10.2.2 and 4.10.4.2.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:
Chandu P. Patel, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-382

Enclosures: 1. Amendment No.136 to NPF-38
2. Safety Evaluation

cc w/encls: See next page

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Document Name: WAT95370.AMD

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 20, 1997

Mr. Charles M. Dugger
Vice President Operations
Entergy Operations, Inc.
P. O. Box B
Killona, LA 70066

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Dear Mr. Dugger:

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The amendment changes the Appendix A TSs by correcting several inconsistencies. These changes effect Table 3.3-4, the allowable values of the parameters; Table 4.3-2 Item 5.c, Safety Injection System Automatic Actuation Logic, modes for which the surveillances are required; and TS 4.10.2.2 and 4.10.4.2.

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Sincerely,

Chandu P. Patel
Chandu P. Patel, Project Manager
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Docket No. 50-382

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Mr. Charles M. Dugger
Entergy Operations, Inc.

Waterford 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 136
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated February 6, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 136, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Chandu P. Patel

Chandu P. Patel, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 20, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 136

TO FACILITY OPERATING LICENSE NO. NPF-38

DOCKET NO. 50-382

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES

3/4 3-19
3/4 3-20
3/4 3-26
3/4 10-2
3/4 10-4

INSERT PAGES

3/4 3-19
3/4 3-20
3/4 3-26
3/4 10-2
3/4 10-4

TABLE 3.3-4

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>	
1. SAFETY INJECTION (SIAS)			
a. Manual (Trip Buttons)	Not Applicable	Not Applicable	
b. Containment Pressure - High	≤ 17.1 psia	≤ 17.4 psia	
c. Pressurizer Pressure - Low	≥ 1684 psia ⁽¹⁾	≥ 1649.7 psia ⁽¹⁾	
d. Automatic Actuation Logic	Not Applicable	Not Applicable	(
2. CONTAINMENT SPRAY (CSAS)			
a. Manual (Trip Buttons)	Not Applicable	Not Applicable	
b. Containment Pressure -- High-High	≤ 17.7 psia	≤ 218.0 psia	
c. Automatic Actuation Logic	Not Applicable	Not Applicable	
3. CONTAINMENT ISOLATION (CIAS)			
a. Manual CIAS (Trip Buttons)	Not Applicable	Not Applicable	
b. Containment Pressure - High	≤ 17.1 psia	≤ 17.4 psia	
c. Pressurizer Pressure - Low	≥ 1684 psia ⁽¹⁾	≥ 1649.7 psia ⁽¹⁾	
d. Automatic Actuation Logic	Not Applicable	Not Applicable	(
4. MAIN STEAM LINE ISOLATION			
a. Manual (Trip Buttons)	Not Applicable	Not Applicable	
b. Steam Generator Pressure - Low	≥ 764 psia ⁽²⁾	≥ 749.9 psia ⁽²⁾	
c. Containment Pressure - High	≤ 17.1 psia	≤ 17.4 psia	
d. Automatic Actuation Logic	Not Applicable	Not Applicable	

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES

<u>FUNCTIONAL UNIT</u>	<u>TRIP VALUE</u>	<u>ALLOWABLE VALUES</u>
5. SAFETY INJECTION SYSTEM SUMP RECIRCULATION (RAS)		
a. Manual RAS (Trip Buttons)	Not Applicable	Not Applicable
b. Refueling Water Storage Pool - Low	10.0% (57,967 gallons)	9.08% (52,634 gallons)
c. Automatic Actuation Logic	Not Applicable	Not Applicable
6. LOSS OF POWER		
a. 4.16 kV Emergency Bus Undervoltage (Loss of Voltage)	≥ 3245 volts	≥ 3245 volts
b. 480 V Emergency Bus Undervoltage	≥ 372 volts	≥ 354 volts
c. 4.16 kV Emergency Bus Undervoltage (Degraded Voltage)	≥ 3875 volts	≥ 3860 volts
7. EMERGENCY FEEDWATER (EFAS)		
a. Manual (Trip Buttons)	Not Applicable	Not Applicable
b. Steam Generator (1&2) Level - Low	$\geq 27.4\%^{(3) (4)}$	$\geq 26.48\%^{(3) (4)}$
c. Steam Generator ΔP - High (SG-1 > SG-2) ≤ 123 psid		≤ 134 psid
d. Steam Generator ΔP - High (SG-2 > SG-1) ≤ 123 psid		≤ 134 psid
e. Steam Generator (1&2) Pressure - Low ≥ 764 psia ⁽²⁾		≥ 749.9 psia ⁽²⁾
f. Automatic Actuation Logic	Not Applicable	Not Applicable
g. Control Valve Logic (Wide Range SG Level - Low)	$\geq 36.3\%^{(3) (5)}$	$\geq 35.3\%^{(3) (5)}$

TABLE 4.3-2

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. SAFETY INJECTION (SIAS)				
a. Manual (Trip Buttons)	N.A.	N.A.	R	1, 2, 3, 4
b. Containment Pressure - High	S	R	Q	1, 2, 3
c. Pressurizer Pressure - Low	S	R	Q	1, 2, 3
d. Automatic Actuation Logic (except subgroup relays)	N.A.	N.A.	Q(2)	1, 2, 3
Actuation Subgroup Relays	N.A.	N.A.	M(3) (6)	1, 2, 3
2. CONTAINMENT SPRAY (CSAS)				
a. Manual (Trip Buttons)	N.A.	N.A.	R	1, 2, 3, 4
b. Containment Pressure -- High - High	S	R	Q	1, 2, 3
c. Automatic Actuation Logic (except subgroup relays)	N.A.	N.A.	Q(2)	1, 2, 3
Actuation Subgroup Relays	N.A.	N.A.	M(1) (3)	1, 2, 3
3. CONTAINMENT ISOLATION (CIAS)				
a. Manual CIAS (Trip Buttons)	N.A.	N.A.	R	1, 2, 3, 4
b. Containment Pressure - High	S	R	Q	1, 2, 3
c. Pressurizer Pressure - Low	S	R	Q	1, 2, 3
d. Automatic Actuation Logic (except subgroup relays)	N.A.	N.A.	Q(2)	1, 2, 3
Actuation Subgroup Relays	N.A.	N.A.	M(1) (3)	1, 2, 3
4. MAIN STEAM LINE ISOLATION				
a. Manual (Trip Buttons)	N.A.	N.A.	R	1, 2, 3
b. Steam Generator Pressure - Low	S	R	Q	1, 2, 3
c. Containment Pressure - High	S	R	Q	1, 2, 3
d. Automatic Actuation Logic (except subgroup relays)	N.A.	N.A.	Q(2)	1, 2, 3
Actuation Subgroup Relays	N.A.	N.A.	M(1) (3)	1, 2, 3

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
5. SAFETY INJECTION SYSTEM RECIRCULATION (RAS)				
a. Manual RAS (Trip Buttons)	N.A.	N.A.	R	1, 2, 3, 4
b. Refueling Water Storage Pool - Low	S	R	Q	1, 2, 3, 4
c. Automatic Actuation Logic (except subgroup relays)	N.A.	N.A.	Q(2)	1, 2, 3, 4
Actuation Subgroup Relays	N.A.	N.A.	M(1) (3)	1, 2, 3, 4
6. LOSS OF POWER (LOV)				
a. 4.16 kV Emergency Bus Undervoltage (Loss of Voltage)	N.A.	R	D(4)	1, 2, 3
b. 480 V Emergency Bus Undervoltage (Loss of Voltage)	N.A.	R	D(4)	1, 2, 3
c. 4.16 kV Emergency Bus Undervoltage (Degraded Voltage)	N.A.	R	D(4)	1, 2, 3

3/4.10 SPECIAL TEST EXCEPTIONS

3/4.10.1 SHUTDOWN MARGIN

LIMITING CONDITION FOR OPERATION

3.10.1 The SHUTDOWN MARGIN requirement of Specification 3.1.1.1 or 3.1.1.2 may be suspended for measurement of CEA worth and SHUTDOWN MARGIN provided reactivity equivalent to at least the highest estimated CEA worth is available for trip insertion from OPERABLE CEA(s).

APPLICABILITY: MODES 2 AND 3*.

ACTION:

- a. With any full-length CEA not fully inserted and with less than the above reactivity equivalent available for trip insertion, immediately initiate and continue boration at greater than or equal to 40 gpm of a solution containing greater than or equal to 1720 ppm boron or its equivalent until the SHUTDOWN MARGIN required by Specification 3.1.1.1 is restored.
- b. With all full-length CEAs fully inserted and the reactor subcritical by less than the above reactivity equivalent, immediately initiate and continue boration at greater than or equal to 40 gpm of a solution containing greater than or equal to 1720 ppm boron or its equivalent until the SHUTDOWN MARGIN required by Specification 3.1.1.2 is restored.

SURVEILLANCE REQUIREMENTS

4.10.1.1 The position of each full-length and part-length CEA required either partially or fully withdrawn shall be determined at least once per 2 hours.

4.10.1.2 Each CEA not fully inserted shall be demonstrated capable of full insertion when tripped from at least the 50% withdrawn position within 7 days prior to reducing the SHUTDOWN MARGIN to less than the limits of Specification 3.1.1.1.

*Operation in MODE 3 shall be limited to 6 consecutive hours.

SPECIAL TEST EXCEPTIONS

3/4.10.2 MODERATOR TEMPERATURE COEFFICIENT, GROUP HEIGHT, INSERTION, AND POWER DISTRIBUTION LIMITS

LIMITING CONDITION FOR OPERATION

3.10.2 The moderator temperature coefficient, group height, insertion, and power distribution limits of Specifications 3.1.1.3, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.1.3.7, 3.2.2, 3.2.3, 3.2.7, and the Minimum Channels OPERABLE requirement of Functional Unit 15 of Table 3.3-1 may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER is restricted to the test power plateau which shall not exceed 85% of RATED THERMAL POWER, and
- b. The limits of Specification 3.2.1 are maintained and determined as specified in Specification 4.10.2.2 below.

APPLICABILITY: MODES 1 and 2.

ACTION:

With any of the limits of Specification 3.2.1 being exceeded while the requirements of Specifications 3.1.1.3, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.1.3.7, 3.2.2, 3.2.3, 3.2.7, and the Minimum Channels OPERABLE requirement of Functional Unit 15 of Table 3.3-1 are suspended, either:

- a. Reduce THERMAL POWER sufficiently to satisfy the requirements of Specification 3.2.1, or
- b. Be in HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS

4.10.2.1 The THERMAL POWER shall be determined at least once per hour during PHYSICS TESTS in which the requirements of Specifications 3.1.1.3, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.1.3.7, 3.2.2, 3.2.3, 3.2.7, or the Minimum Channels OPERABLE requirement of Functional Unit 15 of Table 3.3-1 are suspended and shall be verified to be within the test power plateau.

4.10.2.2 The linear heat rate shall be determined to be within the limits of Specification 3.2.1 by monitoring it continuously with the Incore Detection Monitoring System pursuant to the requirements of Specifications 4.2.1.2 during PHYSICS TESTS above 5% of RATED THERMAL POWER in which the requirements of Specifications 3.1.1.3, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.1.3.7, 3.2.2, 3.2.3, 3.2.7, or the Minimum Channels OPERABLE requirement of Functional Unit 15 of Table 3.3-1 are suspended.

SPECIAL TEST EXCEPTIONS

3/4.10.3 REACTOR COOLANT LOOPS

LIMITING CONDITION FOR OPERATION

3.10.3 The noted requirements of Tables 2.2-1 and 3.3-1 may be suspended during the performance of startup and PHYSICS TESTS, provided:

- a. The THERMAL POWER does not exceed 5% of RATED THERMAL POWER, and either
- b. The reactor trip setpoints of the OPERABLE power level channels are set at less than or equal to 20% of RATED THERMAL POWER, or
- c. The core protection calculator operating bypass permissive setpoints are increased to greater than the logarithmic power-hi trip setpoint specified in Table 2.2-1 and less than 5% RATED THERMAL POWER.

APPLICABILITY: During startup and PHYSICS TESTS.

ACTION:

With the THERMAL POWER greater than 5% of RATED THERMAL POWER, immediately trip the reactor.

SURVEILLANCE REQUIREMENTS

4.10.3.1 The THERMAL POWER shall be determined to be less than or equal to 5% of RATED THERMAL POWER at least once per hour during startup and PHYSICS TESTS.

4.10.3.2 Each wide range logarithmic and power level neutron flux monitoring channel shall be subjected to a CHANNEL FUNCTIONAL TEST within 12 hours prior to initiating startup and PHYSICS TESTS.

SPECIAL TEST EXCEPTIONS

3/4.10.4 CENTER CEA MISALIGNMENT

LIMITING CONDITION FOR OPERATION

3.10.4 The requirements of Specifications 3.1.3.1 and 3.1.3.6 may be suspended during the performance of PHYSICS TESTS to determine the isothermal temperature coefficient, moderator temperature coefficient, and power coefficient provided:

- a. Only the center CEA (CEA #1) is misaligned, and
- b. The limits of Specification 3.2.1 are maintained and determined as specified in Specification 4.10.4.2 below.

APPLICABILITY: MODES 1 and 2.

ACTION:

With any of the limits of Specification 3.2.1 being exceeded while the requirements of Specifications 3.1.3.1 and 3.1.3.6 are suspended, either:

- a. Reduce THERMAL POWER sufficiently to satisfy the requirements of Specification 3.2.1, or
- b. Be in HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS

4.10.4.1 The THERMAL POWER shall be determined at least once per hour during PHYSICS TESTS in which the requirements of Specifications 3.1.3.1 and/or 3.1.3.6 are suspended and shall be verified to be within the test power plateau.

4.10.4.2 The linear heat rate shall be determined to be within the limits of Specification 3.2.1 by monitoring it continuously with the Incore Detection Monitoring System pursuant to the requirements of Specifications 4.2.1.2 during PHYSICS TESTS above 5% of RATED THERMAL POWER in which the requirements of Specifications 3.1.3.1 and/or 3.1.3.6 are suspended.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 136 TO

FACILITY OPERATING LICENSE NO. NPF-38

ENTERGY OPERATIONS, INC.

WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By application dated February 6, 1996, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Waterford Steam Electric Station, Unit 3, Technical Specifications (TSs). The requested changes would modify the TS to correct several inconsistencies.

- a) Revise the following allowable values for parameters identified in Table 3.3-4, "Engineering Safety Features Actuation System Instrumentation Trip Values," to be consistent with Table 2.2-1, "Reactor Protection Instrumentation Trip Setpoint Limits".

	<u>Old Value</u>	<u>New Value</u>
Containment Pressure - High	≤ 17.3 psia	≤ 17.4 psia
Pressurizer Pressure - Low	≥ 1644 psia	≥ 1649.9 psia
Steam Generator Pressure - Low	≥ 748 psia	≥ 749.9 psia
Steam Generator Level - Low	$\geq 26.7\%$	$\geq 26.48\%$

- b) Revise TS 4.10.2.2 and TS 4.10.4.2 of section 3/4.10 Special Test Exceptions to delete reference to previously removed TS 3.3.3.2.
- c) Revise Table 4.3-2, Item 5.c. to specify Mode 4 applicability to be consistent with TS Table 3.3-3.

2.0 EVALUATION

- a) On September 5, 1995, Amendment 113 to Facility Operating License NPF-38 revised several allowable values for parameters identified in Table 2.2-1, "Reactor Protective Instrumentation Trip Setpoint Limits" so that they would be consistent with the setpoint/uncertainty methodology. The licensee's application dated December 9, 1994, requesting the proposed change failed to identify the duplicate parameters and values appearing in Table 3.3-4, "Engineering Safety Features Actuation System Instrumentation Trip Values". Since this change is requesting that the allowable values

for parameters in Table 3.3-4 be consistent with Table 2.2-1, the change is administrative. The technical basis for revising the values is provided in the staff's safety evaluation for Amendment 113.

- b) On May 30, 1995, Amendment 107 to Facility Operating License NPF-38 removed the Incore Detection System Requirements specified in TS 3.3.3.2 and deleted TS 3.3.3.2. The basis for moving the incore detector requirements from the TS to the UFSAR is provided in the staff's safety evaluation for Amendment 107. This change is only deleting reference in TS 4.10.2.2 and TS 4.10.4.2 to the previously deleted TS 3.3.3.2 and is administrative.
- c) On November 9, 1992, Amendment 78 to Facility Operating License NPF-38 inadvertently deleted Mode 4 from specified surveillance requirements of TS Table 4.3-2, Item 5.c. Revising Table 4.3-2, Item 5.c. to specify Mode 4 applicability to be consistent with TS Table 3.3-3 is an administrative change to correct this earlier error.

The staff has concluded that all of the above changes to the TS to improve consistency and to correct previous inadvertent errors are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding 61 FR 28615. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Marsha Gamberoni

Date: November 20, 1997