

Mr. Charles M. Dugger
 Vice President Operations
 Entergy Operations, Inc.
 P. O. Box B
 Killona, LA 70066

April 1, 1998

SUBJECT: ISSUANCE OF AMENDMENT NO.140 TO FACILITY OPERATING LICENSE
 NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3 (TAC NO.
 M97830)

Dear Mr. Dugger:

The Commission has issued the enclosed Amendment No. 140 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 18, 1996, as supplemented by letter dated January 21, 1998.

The amendment changes the Appendix A TSs by changing TS Surveillance Requirement 4.4.8.3.1.b to test the Shutdown Cooling System suction line relief valves in accordance with TS 4.0.5. Editorial changes to TS 4.4.8.3.1 and 4.4.8.3.1.a. have also been made.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
 ORIGINAL SIGNED BY:
 Chandu P. Patel, Project Manager
 Project Directorate IV-1
 Division of Reactor Projects III/IV
 Office of Nuclear Reactor Regulation

Docket No. 50-382

Enclosures: 1. Amendment No. 140 to NPF-38
 2. Safety Evaluation

NO FILE REQUIRED COPY

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 1, 1998

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P. O. Box B
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Chandu P. Patel
Chandu P. Patel, Project Manager
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Mr. Charles M. Dugger
Entergy Operations, Inc.

Waterford 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated November 18, 1996, as supplemented by letter dated January 21, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 140 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Chandu P. Patel

Chandu P. Patel, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: April 1, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 140

TO FACILITY OPERATING LICENSE NO. NPF-38

DOCKET NO. 50-382

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE PAGE

3/4 4-35

INSERT PAGE

3/4 4-35

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.8.3.1 For each SDC System suction line relief valve:

- a. verify in the control room at least once per 12 hours that each valve in the suction path between the RCS and the SDC relief valve is open.
- b. verify each SDC relief valve is OPERABLE in accordance with TS 4.0.5.

4.4.8.3.2 With the RCS vented per ACTIONS a, b, or c, the RCS vent(s) and all valves in the vent path shall be verified to be open at least once per 12 hours*.

*Except when the vent pathway is provided with a valve which is locked, sealed, or otherwise secured in the open position, then verify these valves open at least once per 31 days.

REACTOR COOLANT SYSTEM

3/4.4.9 STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.9 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.9.

APPLICABILITY: All MODES.

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 70°F above the minimum temperature required by NDT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F, except during hydrostatic testing of components that are nonisolable from the Reactor Coolant System, then restore the structural integrity prior to increasing the Reactor Coolant System temperature more than 30°F above the minimum temperature required by NDT considerations.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component to within its limit or isolate the affected component from service.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.9 In addition to the requirements of Specification 4.0.5, each reactor coolant pump flywheel shall be inspected per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 140 TO

FACILITY OPERATING LICENSE NO. NPF-38

ENERGY OPERATIONS, INC.

WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By application dated November 18, 1996, as supplemented by letter dated January 21, 1998, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Waterford Steam Electric Station, Unit 3 (Waterford 3), Technical Specifications (TSs). The requested changes would change TS Surveillance Requirement 4.4.8.3.1.b to test the Shutdown Cooling System suction line relief valves in accordance with TS 4.0.5. Editorial changes to TS 4.4.8.3.1 and 4.4.8.3.1.a. have also been requested.

2.0 EVALUATION

The Shutdown Cooling System (SDCS) suction line relief valves at Waterford 3 are a direct acting, spring loaded, closed bonnet design. The relief valve protects the system from inadvertent Reactor Coolant System pressurization during SDCS operation when the suction line isolation valves are open.

Waterford 3 TS currently require testing of each valve every 30 months. Eleven years of historical data regarding these tests indicate that, when bench tested, the valves have been within lift pressure (< 430 psia) with two exceptions. Only one of the failures was due to drift in valve setpoint.

Waterford 3 is on an 18 month fuel cycle, therefore both valves require testing during each refueling outage. The November 18, 1996, submittal requests a testing interval specified by the applicable ASME code referenced in TS 4.0.5. Changing the test frequency to test in accordance with TS 4.0.5 will allow testing of one valve each refueling outage by applying a staggered-test basis.

The Combustion Engineering Standard Technical Specification Bases specify a frequency of 18 months for power operated relief valves based on a typical refueling cycle and industry accepted practice, while the CE STS Bases specify that the testing frequency for mechanical relief valves should be in accordance with the requirements of Section XI of the ASME Code.

Based on the SDCS suction line relief valves being mechanical relief valves and the reliable as-found setpoint data discussed above, the proposed change in testing frequency as stated in the revised TS is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comment.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 6985). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Gamberoni

Date: April 1, 1998