

September 5, 1995

Mr. Ross P. Barkhurst
Vice President Operations
Entergy Operations, Inc.
P. O. Box B
Killona, LA 70066

SUBJECT: ISSUANCE OF AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE
NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3 (TAC NO. M89773)

Dear Mr. Barkhurst:

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated June 22, 1994.

The amendment changes the Appendix A Technical Specifications by removing the seismic and meteorological monitoring instrumentation requirements. These requirements are to be relocated in the Updated Final Safety Analysis Report.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By:

Chandu P. Patel, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

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Docket No. 50-382

Enclosures: 1. Amendment No. 112 to NPF-38
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 5, 1995

Mr. Ross P. Barkhurst
Vice President Operations
Entergy Operations, Inc.
P. O. Box B
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Chandu P. Patel

Chandu P. Patel, Project Manager
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Docket No. 50-382

Enclosures: 1. Amendment No. 112 to NPF-38
2. Safety Evaluation

cc w/encls: See next page

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Waterford 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated June 22, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 112, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Chandu P. Patel

Chandu P. Patel, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: September 5, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 112
TO FACILITY OPERATING LICENSE NO. NPF-38
DOCKET NO. 50-382

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

REMOVE PAGES

V
XX
XXI
3/4 3-35
3/4 3-36
3/4 3-37
3/4 3-38
3/4 3-39
3/4 3-40
B 3/4 3-2

INSERT PAGES

V
XX
XXI
3/4 3-35
-
-
-
-
-
B 3/4 3-2

INDEX

LIMITING CONDITION FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 LINEAR HEAT RATE.....	3/4 2-1
3/4.2.2 PLANAR RADIAL PEAKING FACTORS.....	3/4 2-3
3/4.2.3 AZIMUTHAL POWER TILT.....	3/4 2-4
3/4.2.4 DNBR MARGIN.....	3/4 2-6
3/4.2.5 RCS FLOW RATE.....	3/4 2-10
3/4.2.6 REACTOR COOLANT COLD LEG TEMPERATURE.....	3/4 2-11
3/4.2.7 AXIAL SHAPE INDEX.....	3/4 2-12
3/4.2.8 PRESSURIZER PRESSURE.....	3/4 2-13
 <u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR PROTECTIVE INSTRUMENTATION.....	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-13
3/4.3.3 MONITORING INSTRUMENTATION	
RADIATION MONITORING INSTRUMENTATION.....	3/4 3-28
REMOTE SHUTDOWN INSTRUMENTATION.....	3/4 3-41
ACCIDENT MONITORING INSTRUMENTATION.....	3/4 3-44
CHEMICAL DETECTION SYSTEMS.....	3/4 3-47
EXPLOSIVE GAS MONITORING INSTRUMENTATION.....	3/4 3-60
3/4.3.4 DELETED.....	3/4 3-68

INDEX

LIMITING CONDITION FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION	
STARTUP AND POWER OPERATION.....	3/4 4-1
HOT STANDBY.....	3/4 4-2
HOT SHUTDOWN.....	3/4 4-3
COLD SHUTDOWN - LOOPS FILLED.....	3/4 4-5
COLD SHUTDOWN - LOOPS NOT FILLED.....	3/4 4-6
3/4.4.2 SAFETY VALVES	
SHUTDOWN.....	3/4 4-7
OPERATING.....	3/4 4-8
3/4.4.3 PRESSURIZER	
PRESSURIZER.....	3/4 4-9
AUXILIARY.....	3/4 4-9a
3/4.4.4 STEAM GENERATORS.....	3/4 4-10
3/4.4.5 REACTOR COOLANT SYSTEM LEAKAGE	
LEAKAGE DETECTION SYSTEMS.....	3/4 4-17
OPERATIONAL LEAKAGE.....	3/4 4-18
3/4.4.6 CHEMISTRY.....	3/4 4-21
3/4.4.7 SPECIFIC ACTIVITY.....	3/4 4-24
3/4.4.8 PRESSURE/TEMPERATURE LIMITS	
REACTOR COOLANT SYSTEM.....	3/4 4-28
PRESSURIZER HEATUP/COOLDOWN.....	3/4 4-33
OVERPRESSURE PROTECTION SYSTEMS.....	3/4 4-34
3/4.4.9 STRUCTURAL INTEGRITY.....	3/4 4-36
3/4.4.10 REACTOR COOLANT SYSTEM VENTS.....	3/4 4-37
<u>3/4.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)</u>	
3/4.5.1 SAFETY INJECTION TANKS.....	3/4 5-1
3/4.5.2 ECCS SUBSYSTEMS - Modes 1, 2, and 3	3/4 5-3
3/4.5.3 ECCS SUBSYSTEMS - Modes 3 and 4	3/4 5-8
3/4.5.4 REFUELING WATER STORAGE POOL.....	3/4 5-9

INDEX

LIST OF TABLES

<u>TABLE</u>		<u>PAGE</u>
1.1	FREQUENCY NOTATION.....	1-9
1.2	OPERATIONAL MODES.....	1-10
2.2-1	REACTOR PROTECTIVE INSTRUMENTATION TRIP SETPOINT LIMITS.....	2-3
2.2-2	CORE PROTECTION CALCULATOR ADDRESSABLE CONSTANTS..... MONITORING FREQUENCIES FOR BORON DILUTION DETECTION	2-5
3.3-1	REACTOR PROTECTIVE INSTRUMENTATION.....	3/4 3-3
4.3-1	REACTOR PROTECTIVE INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-10
3.3-3	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-14
3.3-4	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES.....	3/4 3-19
4.3-2	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-25
3.3-6	RADIATION MONITORING INSTRUMENTATION.....	3/4 3-29
4.3-3	RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-32

INDEX

LIST OF FIGURES

<u>FIGURE</u>		<u>PAGE</u>
3.1-1	REQUIRED STORED BORIC ACID VOLUME AS A FUNCTION OF CONCENTRATION.....	3/4 1-13
3.4-1	DOSE EQUIVALENT I-131 PRIMARY COOLANT SPECIFIC ACTIVITY LIMIT VERSUS PERCENT OF RATED THERMAL POWER WITH THE PRIMARY COOLANT SPECIFIC ACTIVITY >1.0 μ Ci/GRAM DOSE EQUIVALENT I-131.....	3/4 4-27
3.4-2	REACTOR COOLANT SYSTEM PRESSURE/TEMPERATURE LIMITATIONS FOR 0-8 EFFECTIVE FULL POWER YEARS (HEATUP).....	3/4 4-30
3.4-3	REACTOR COOLANT SYSTEM PRESSURE/TEMPERATURE LIMITATIONS FOR 0-8 EFFECTIVE FULL POWER YEARS (COOLDOWN).....	3/4 4-31
4.7-1	SAMPLING PLAN FOR SNUBBER FUNCTIONAL TEST.....	3/4 7-26
5.1-1	EXCLUSION AREA.....	5-2
5.1-2	LOW POPULATION ZONE.....	5-3
5.1-3	SITE BOUNDARY FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS.....	5-4
6.2-1	OFFSITE ORGANIZATION FOR MANAGEMENT AND TECHNICAL SUPPORT.....	6-3
6.2-2	PLANT OPERATIONS ORGANIZATION.....	6-4

INDEX

LIST OF TABLES (Continued)

<u>TABLE</u>	<u>PAGE</u>
3.3-9 REMOTE SHUTDOWN INSTRUMENTATION.....	3/4 3-42
4.3-6 REMOTE SHUTDOWN INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-43
3.3-10 ACCIDENT MONITORING INSTRUMENTATION.....	3/4 3-45
4.3-7 ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-46
3.3-12 DELETED.....	3/4 3-56
4.3-8 DELETED.....	3/4 3-58
3.3-13 EXPLOSIVE GAS MONITORING INSTRUMENTATION.....	3/4 3-61
4.3-9 EXPLOSIVE GAS MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-65
4.4-1 MINIMUM NUMBER OF STEAM GENERATORS TO BE INSPECTED DURING INSERVICE INSPECTION.....	3/4 4-15
4.4-2 STEAM GENERATOR TUBE INSPECTION.....	3/4 4-16
3.4-1 REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES.....	3/4 4-20
3.4-2 REACTOR COOLANT SYSTEM CHEMISTRY.....	3/4 4-22
4.4-3 REACTOR COOLANT SYSTEM CHEMISTRY LIMITS SURVEILLANCE REQUIREMENTS.....	3/4 4-23
4.4-4 PRIMARY COOLANT SPECIFIC ACTIVITY SAMPLE AND ANALYSIS PROGRAM.....	3/4 4-26

INDEX

LIST OF TABLES (Continued)

<u>TABLE</u>		<u>PAGE</u>
3.7-1	STEAM LINE SAFETY VALVES PER LOOP.....	3/4 7-2
3.7-2	MAXIMUM ALLOWABLE LINEAR POWER LEVEL - HIGH TRIP SETPOINT WITH INOPERABLE STEAM LINE SAFETY VALVES DURING OPERATION WITH BOTH STEAM GENERATORS.....	3/4 7-3
4.7-1	SECONDARY COOLANT SYSTEM SPECIFIC ACTIVITY SAMPLE AND ANALYSIS PROGRAM.....	3/4 7-8
3.7-3	ULTIMATE HEAT SINK MINIMUM FAN REQUIREMENTS.....	3/4 7-14
4.8-1	DIESEL GENERATOR TEST SCHEDULE.....	3/4 8-7
4.8-1a	ADDITIONAL RELIABILITY ACTIONS.....	3/4 8-7a
4.8-2	BATTERY SURVEILLANCE REQUIREMENTS.....	3/4 8-11

PAGES 3/4 3-36
THROUGH
PAGE 3/4 3-40
NOT USED

INSTRUMENTATION

BASES

individual channels; (2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded; and (3) sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," December 1980 and NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980.

3/4.3.3.2 INCORE DETECTORS

This section has been deleted.

3/4.3.3.3 SEISMIC INSTRUMENTATION

This section has been deleted.

3/4.3.3.4. METEOROLOGICAL INSTRUMENTATION

This section has been deleted.

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criterion 19 of 10 CFR Part 50.

3/4.3 INSTRUMENTATION

BASES

3/4.3.1 and 3/4.3.2 REACTOR PROTECTIVE AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEMS INSTRUMENTATION

The OPERABILITY of the Reactor Protective and Engineered Safety Features Actuation Systems instrumentation and bypasses ensures that (1) the associated Engineered Safety Features Actuation action and/or reactor trip will be initiated when the parameter monitored by each channel or combination thereof reaches its setpoint, (2) the specified coincidence logic is maintained, (3) sufficient redundancy is maintained to permit a channel to be out of service for testing or maintenance, and (4) sufficient system functional capability is available from diverse parameters.

The OPERABILITY of these systems is required to provide the overall reliability, redundancy, and diversity assumed available in the facility design for the protection and mitigation of accident and transient conditions. The integrated operation of each of these systems is consistent with the assumptions used in the safety analyses.

The redundancy design of the Control Element Assembly Calculators (CEAC) provides reactor protection in the event one or both CEACs become inoperable. If one CEAC is in test or inoperable, verification of CEA position is performed at least every 4 hours. If the second CEAC fails, the CPCs will use DNBR and LPD penalty factors to restrict reactor operation to some maximum fraction of RATED THERMAL POWER. If this maximum fraction is exceeded, a reactor trip will occur.

The Surveillance Requirements specified for these systems ensure that the overall system functional capability is maintained comparable to the original design standards. The periodic surveillance tests performed at the minimum frequencies are sufficient to demonstrate this capability. The quarterly frequency for the channel functional tests for these systems comes from the analyses presented in topical report CEN-327: RPS/ESFAS Extended Test Interval Evaluation, as supplemented.

The measurement of response time at the specified frequencies provides assurance that the protective and ESF action function associated with each channel is completed within the time limit assumed in the safety analyses. No credit was taken in the analyses for those channels with response times indicated as not applicable.

Response time may be demonstrated by any series of sequential, overlapping, or total channel test measurements provided that such tests demonstrate the total channel response time as defined. Sensor response time verification may be demonstrated by either (1) in place, onsite, or offsite test measurements or (2) utilizing replacement sensors with certified response times.

3/4.3.3 MONITORING INSTRUMENTATION

3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that: (1) the radiation levels are continually measured in the areas served by the



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 112 TO

FACILITY OPERATING LICENSE NO. NPF-38

ENTERGY OPERATIONS, INC.

WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By application dated June 22, 1994, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Waterford Steam Electric Station, Unit 3, Technical Specifications (TSs). The requested changes would remove the seismic and meteorological instrumentation requirements from the TSs. The requirements are to be included in the updated final safety analysis report (UFSAR) and controlled through 10 CFR 50.59.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act, as amended (the "Act") requires that applicants for nuclear power plant operating licenses incorporate TS as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

The Commission has provided guidance for the contents of TSs in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies § 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TSs to licensee-controlled documents, consistent with the standard enunciated in *Portland General Electric Co.* (Trojan Nuclear Plant), ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TS, as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; and (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. As a result, existing TS requirements which fall within or satisfy any of the criteria in the Final Policy Statement must be retained in the TSs, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

The Commission issued a change to 10 CFR 50.36, 60 FR 36959 (July 19, 1995), pursuant to which the rule was amended to codify and incorporate the guidance contained in the Final Policy Statement.

3.0 EVALUATION

Seismic Monitoring Instrumentation

Section VI(a)(3) of Appendix A to 10 CFR Part 100 requires that seismic monitoring instrumentation be provided to promptly determine the response of those nuclear power plant features important to safety in the event of an earthquake. This capability is required to allow for a comparison of the measured response to that used in the design basis for the unit. Comparison of such data is needed to (1) determine whether the plant can continue to be operated safely, and (2) permit such timely action as may be appropriate. However, seismic instrumentation does not actuate any protective equipment or serve any direct role in the mitigation of an accident.

The capability of the plant to withstand a seismic event or other design-basis accident is determined by the initial design and construction of systems, structures, and components. The instrumentation is used to alert operators to the seismic event and evaluate the plant response. The Final Policy Statement explained that instrumentation to detect precursors to reactor coolant pressure boundary leakage, such as seismic instrumentation, is not included in the first criterion. As discussed above, the seismic instrumentation does not serve as a protective design feature or part of a primary success path for events which challenge fission product barriers. The staff has concluded that the seismic monitoring instrumentation does not satisfy the final policy statement criteria and need not be included in the TS. The licensee has proposed to relocate the seismic monitoring instrumentation requirements to the UFSAR and control changes to those provisions in accordance with 10 CFR 50.59.

Meteorological Monitoring Instrumentation

In 10 CFR 50.47, "Emergency Plans," and 10 CFR Part 50, Appendix E, "Emergency Planning and Preparedness for Production and Utilization Facilities," the Commission requires power plant licensees to provide reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency. Timely access to accurate local meteorological data is important for estimating potential radiation doses to the public and for determining appropriate protective measures. In 10 CFR 50.36a(a)(2), the Commission requires nuclear power plant licensees to submit annual reports specifying the quantity of each of the principal radionuclides released to unrestricted areas in liquid and airborne effluents and such other information as may be required by the NRC to estimate maximum potential annual radiation doses to the public. A knowledge of meteorological conditions in the vicinity of the reactor is important in providing a basis for estimating annual radiation doses resulting from radioactive materials released in airborne effluents. Accordingly, the meteorological monitoring instrumentation serves a useful function in estimating radiation doses to the public from either routine or accidental releases of radioactive materials to the atmosphere.

The meteorological monitoring instrumentation does not serve such a primary protective function as to warrant inclusion in the TS in accordance with the criteria of the final policy statement. The instrumentation does not serve to ensure that the plant is operated within the bounds of initial conditions assumed in design basis accident and transient analyses or that the plant will be operated to preclude transients or accidents. Likewise, the meteorological instrumentation does not serve as part of the primary success path of a safety sequence analysis used to demonstrate that the consequences of these events are within the appropriate acceptance criteria. Accordingly, the staff has concluded that the meteorological instrumentation does not satisfy the final policy statement criteria and need not be included in TS. The staff has determined that requirements related to the meteorological monitoring instrumentation can be moved from the TS to the UFSAR, and that any subsequent changes to the provisions would be controlled pursuant to 10 CFR 50.59.

In conclusion, the above relocated requirements relating to seismic and meteorological monitoring instrumentation are not required to be in the TS under 10 CFR 50.36 or 182a of the Atomic Energy Act, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. Further, they do not fall within any of the four criteria set forth in the Commission's Final Policy Statement, and codified in the revision of 10 CFR 50.36. In addition, the Staff finds that sufficient regulatory controls exist under 10 CFR 50.59 to address any future changes to these systems. Accordingly, the staff has concluded that the proposed change to relocate the seismic and meteorological monitoring instrumentation requirements from the TSs to the UFSAR is acceptable. With this action, the table of contents entry and the BASES section for TS 3/4.3.3.3 and 3/4.3.3.4 may be removed from the TSs.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (59 FR 39585). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Patel

Date: September 5, 1995