

September 27, 1995

Mr. Ross P. Barkhurst  
Vice President Operations  
Entergy Operations, Inc.  
P.O. Box B  
Killona, LA 70066

SUBJECT: ISSUANCE OF AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE  
NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3 (TAC NO. M89775)

Dear Mr. Barkhurst:

The Commission has issued the enclosed Amendment No. 114 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated June 22, 1994, as supplemented by letters dated June 28, 1995 and August 22, 1995.

The amendment changes the Appendix A TSs by increasing the control room radiation monitor setpoint (CRRMS) to a fixed value of 5.45E-6  $\mu$ Ci/cc instead of being set at two times the background.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Timothy Polich/for  
Chandu P. Patel, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-382

Enclosures: 1. Amendment No. 114 to NPF-38  
2. Safety Evaluation

cc w/encls: See next page

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*signed 9/27/95*

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Entergy Operations, Inc.  
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Sincerely,

  
for Chandu P. Patel, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-382

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2. Safety Evaluation

cc w/encls: See next page

Mr. Ross P. Barkhurst  
Entergy Operations, Inc.

Waterford 3

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114  
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated June 22, 1994, as supplemented by letters dated June 28, 1995, and August 22, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*for* 

Chandu P. Patel, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the  
Technical Specifications

Date of Issuance: September 27, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 114

TO FACILITY OPERATING LICENSE NO. NPF-38

DOCKET NO. 50-382

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES

3/4 3-29

INSERT PAGES

3/4 3-29

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area Fuel Handling Building Ventilation System Isolation	2	*	≤ 100 mR/h	10 <sup>-1</sup> - 10 <sup>4</sup> mR/h	24
b. Containment - Purge & Exhaust Isolation	1/train	1, 2, 3, 4 & **	40 mR/h or ≤ 2x background whichever is Higher	20 - 5x10 <sup>5</sup> mR/h	25
2. PROCESS MONITORS					
a. Containment Atmosphere					
1) Gaseous Activity RCS Leakage Detection	1	1, 2, 3, & 4	Not Applicable	10 <sup>-6</sup> - 10 <sup>-1</sup> μCi/cc	23
2) Particulate Activity RCS Leakage Detection	1	1, 2, 3, & 4	Not Applicable	10 <sup>-11</sup> - 10 <sup>-6</sup> μCi/cc	23
b. Control Room Intake Monitors	1/intake	ALL MODES	≤ 4.09x10 <sup>-5</sup> μCi/cc	10 <sup>-8</sup> - 10 <sup>-2</sup> μCi/cc	26
c. Steam Generator Blowdown Monitor	1	1, 2, 3, & 4	≤ 10 <sup>-3</sup> μCi/cc	10 <sup>-6</sup> - 10 <sup>-1</sup> μCi/cc	28
d. Component Cooling Water Monitors A&B	1/line	ALL MODES	≤ 10 <sup>-4</sup> μCi/cc	10 <sup>-7</sup> - 10 <sup>-2</sup> μCi/cc	28
e. Component Cooling Water Monitor A/B	1	1, 2, 3, & 4	≤ 10 <sup>-4</sup> μCi/cc	10 <sup>-7</sup> - 10 <sup>-2</sup> μCi/cc	28

\*With irradiated fuel in the storage pool.

\*\*During CORE ALTERATIONS or movement of irradiated fuel within the containment.

TABLE 3.3-6 (Continued)

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
<b>3. EFFLUENT ACCIDENT MONITORS</b>					
a. Containment High Range	2	1, 2, 3, & 4	Not Applicable	1 - 10 <sup>8</sup> R/h	27
b. Plant Stack High Range	1	1, 2, 3, & 4	Not Applicable	10 <sup>-7</sup> - 10 <sup>5</sup> µCi/cc	27
c. Condenser Vacuum Pump High Range	1	1, 2, 3, & 4	Not Applicable	10 <sup>-7</sup> - 10 <sup>5</sup> µCi/cc	27
d. Fuel Handling Building Exhaust High Range	1	1*, 2*, 3*, & 4*	Not Applicable	10 <sup>-7</sup> - 10 <sup>5</sup> µCi/cc	27
e. Main Steam Line High Range	1/steam line	1, 2, 3, & 4	Not Applicable	1 - 10 <sup>5</sup> mR/h	27

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\*With irradiated fuel in the storage pool.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 114 TO

FACILITY OPERATING LICENSE NO. NPF-38

ENERGY OPERATIONS, INC.

WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By application dated June 22, 1994, as supplemented by letters dated June 28, 1995, and August 22, 1995, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Waterford Steam Electric Station, Unit 3, Technical Specifications (TSs). The requested changes would allow the control room radiation monitor setpoint (CRRMS) to be increased to a fixed value of  $5.45E-6$   $\mu\text{Ci/cc}$  instead of being set at two times the background.

The June 28, 1995, and August 22, 1995, letters provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The staff has completed its evaluation of the potential radiological consequences of the increased radiation monitor setpoint at Waterford 3, based upon the conditions of the proposed TS changes and telephone discussions held with the licensee on July 17, 1995. As committed in the followup letter dated August 22, 1995, the licensee reduced its CRRMS from  $4.09E-5$   $\mu\text{Ci/cc}$  to  $5.45E-6$   $\mu\text{Ci/cc}$ . Based on our review, we conclude that the proposed request to increase the (CRRMS) from two times background to a fixed value of  $5.45E-6$   $\mu\text{Ci/cc}$  will not result in radioactive material concentrations exceeding the control room operator dose guidelines. The staff reviewed the licensee's submittal and, in addition, the staff performed an independent analysis to determine conformance with the requirements of 10 CFR Part 100 and GDC 19 of Appendix A to 10 CFR Part 50. The proposed value is approximately 10 times the current background noise levels and is a factor of 10 below the calculated maximum setpoint value that would prevent radioactive material concentrations in the control room from exceeding the Derived Air Concentration (DAC) occupational values listed in 10 CFR Part 20 Appendix B, Table 1, Column 3.

Based on the staff's review of the licensee's analysis and independent assessment of the radiological consequences of increasing the control room radiation monitor setpoint, the staff concludes that the radiological

consequences associated with this increase are within the acceptance criteria set forth in 10 CFR Part 100 and the control room operator dose criteria specified in GDC-19 of Appendix A to 10 CFR Part 50 and are, therefore, acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (59 FR 39586). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Carter

Date: September 27, 1994