

June 18, 2002

Mr. G. R. Peterson
Site Vice President
Catawba Nuclear Station
Duke Energy Corporation
4800 Concord Road
York, South Carolina 29745-9635

SUBJECT: CATAWBA NUCLEAR STATION, UNIT 2 RE: SUMMARY OF CONFERENCE
CALL WITH DUKE ENERGY REGARDING 2001 STEAM GENERATOR TUBE
INSPECTION RESULTS AT CATAWBA, UNIT 2 (TAC NO. MB3037)

Dear Mr. Peterson:

On October 1 and October 10, 2001, the Nuclear Regulatory Commission (NRC) staff participated in conference calls with representatives of Duke Energy Corporation (Duke) regarding the steam generator tube inspection activities at Catawba, Unit 2 during the refueling outage. A brief summary of the conference calls is provided in Enclosure 1. The material provided by Duke in support of these calls is included in Enclosure 2. It includes the discussion of questions asked by the NRC staff, the steam generator work scope at Catawba, Unit 2 and steam generators tube plugging history for Catawba, Unit 2.

If you have any further questions, please contact me at 301-415-3025.

Sincerely,

/RA/

Chandu P. Patel, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosure: As stated

cc w/encl: See next page

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SUMMARY OF CONFERENCE CALLS
WITH
DUKE ENERGY CORPORATION
REGARDING 2001 STEAM GENERATOR INSPECTION RESULTS
AT CATAWBA, UNIT 2

On October 1 and October 10, 2001, NRC staff participated in conference calls with Duke Energy Corporation (licensee) representatives to discuss the preliminary results of the steam generator tube inspections at Catawba, Unit 2.

On October 1, 2001, the licensee representatives discussed the following topics with regard to the steam generator tube inspection activities at Catawba, Unit 2: initial eddy current testing scope, scope expansion plans, indications identified to date, repair/plugging plans, new inspection findings, in-situ pressure test plans and actions taken in response to lessons learned from the Indian Point 2 tube failure.

The licensee reported that there had not been significant degradation detected. However, the licensee had identified a signal, which had some characteristics of an indication in a tube in row 21, column 75 in steam generator C. The indication was located below the top of the tubesheet but above the expansion transition. The exact location of the indication is 0.04 inch above the expansion transition, and the top of tubesheet is located 0.05 inch above the expansion transition. The licensee initially detected this indication with a plus point coil; however, there was a question as to whether it represented a flaw or deposit. Nonetheless, the licensee subsequently expanded its plus point coil examinations at the top of the tubesheet to 100 percent of the tubing in steam generators A, C, and D.

The licensee performed sludge lancing in steam generator C and used the RG3-4 probe to further characterize the nature of the signal at the top of tubesheet in row 21, column 75. After the sludge lancing, the signal formation had changed from a characteristic of a flaw to that of a small volumetric indication due to deposits. All of the phases for all of the frequencies were outside the defect plane. Based upon these evaluations, the indication was dispositioned as "no detectable degradation." No tubes were plugged at the time of the call.

On October 10, 2001, the licensee representatives discussed in more detail the top of tubesheet signal identified in the tube at row 21, column 75. The licensee also discussed the qualification of the RG3-4 probe and the results provided in steam generator tube inspection reports for prior outages.

On the basis of the information discussed during the calls the staff did not identify any issues requiring further discussion.

By letter dated October 24, 2001 (ML013410150), the licensee submitted a steam generator tube inspection report for the inspection conducted in 2001. The staff will review the report in the near future.

Unit 2 Model D-5 RSG Plugging History

Steam Generator 2A

<u>Date/ RFO</u>	<u>Row</u>	<u>Column</u>	<u>Plug Type</u>	<u>Reason</u>
4/97 EOC 8	18	50	WMPL	Bobbin >40%TW, RPC-NDF
9/98 EOC9	43	68	WMPL	Permeability
3/00 EOC10				No Tubes Repaired
10/01 EOC11				No Tubes Repaired

Steam Generator 2B

<u>Date/ RFO</u>	<u>Row</u>	<u>Column</u>	<u>Plug Type</u>	<u>Reason</u>
4/97 EOC8	29	105	WMPL	MBM/PLP wea
4/97 EOC8	39	85	WMPL	Lack of RPC data
4/97 EOC8	39	97	WMPL	Perm
4/97 EOC8	40	19	WMPL	MBM/PLP wear
4/97 EOC8	43	22	WMPL	MBM/PLP wear
9/98 EOC9	1	61	WMPL	Dent signal change
9/98 EOC9	20	104	WMPL	MBM/PLP wear
9/98 EOC9	35	41	WMPL	MBM/PLP wear
9/98 EOC9	46	54	WMPL	Wear, no size available
9/98 EOC9	48	39	WMPL	Wear, no size available
3/00 EOC10	2	99	WMPL	Plugged due to +point probe lodged in U-bend.
3/00 EOC10	16	29	WMPL	Wear, no size available
3/00 EOC10	34	42	WMPL	Wear, no size available
3/00 EOC10	38	80	WMPL	Plugged for WAR at AVB 2&3 (32%/42%)
10/01 EOC11				No tubes repaired

Steam Generator 2C

<u>Date/ RFO</u>	<u>Row</u>	<u>Column</u>	<u>Plug Type</u>	<u>Reason</u>
4/97 EOC8				No tubes plugged.
9/98 EOC9	32	12	WMPL	AVB wear
3/00 EOC10	1	22	WMPL	No plus point in U-bend
3/00 EOC10	2	30	WMPL	Probe lodged in U-bend
10/01 EOC11				No tubes plugged.

Unit 2 Model D-5 RSG Plugging History (Continued)

Steam Generator 2D

<u>Date/ RFO</u>	<u>Row</u>	<u>Column</u>	<u>Plug Type</u>	<u>Reason</u>
4/97 EOC8	6	81	WMPL	Data Quality
4/97 EOC8	35	93	WMPL	U-bend PVN
4/97 EOC8	38	17	WMPL	AVB Wear > 40%TW
4/97 EOC8	39	18	WMPL	AVB Wear > 40%TW
9/98 EOC9	33	16	WMPL	Wear, no sizing
9/98 EOC9	48	75	WMPL	MBM/PLP Wear
3/00 EOC10	19	65	WMPL	TTS RPC not acquired. Plugged for economics.
10/01 EOC11				No tubes plugged

WMP = Westinghouse IN690 Mech. Plug (short)
WMPL = Westinghouse IN690 Mech. Plug (long)
IN690 = B&W Roll plug
IN600 = B&W Roll Plug

Catawba Nuclear Station

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