Docket No. 50-382

Mr. Ross P. Barkhurst Vice President Operations Entergy Operations, Inc. Post Office Box B Killona. Louisiana 70066

Dear Mr. Barkhurst:

SUBJECT: ISSUANCE OF AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE

NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3 (TAC NO. M83315)

The Commission has issued the enclosed Amendment No. 82 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 24, 1992, as supplemented by letter dated February 25, 1993.

The amendment changes the Appendix A Technical Specifications by removing the requirement to conduct wall-cracking surveillance as part of the basemat cracking program.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly <u>Federal Register</u> notice.

Sincerely,

# ORIGINAL SIGNED BY:

David L. Wigginton, Senior Project Manager Project Directorate IV-1 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

#### Enclosures:

1. Amendment No. 82 to NPF-38

2. Safety Evaluation

cc w/enclosures: See next page

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# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 8, 1993

Docket No. 50-382

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Sincerely,

David L. Wigginton, Senior Project Manager

Project Directorate IV-1

Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

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cc w/enclosures: See next page Mr. Ross P. Barkhurst Entergy Operations, Inc.

#### cc:

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#### Waterford 3

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Chairman Louisiana Public Service Commission One American Place, Suite 1630 Baton Rouge, Louisiana 70825-1697

Mr. R. F. Burski, Director Nuclear Safety Entergy Operations, Inc. P. O. Box B Killona, Louisiana 70066



# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

# ENTERGY OPERATIONS, INC.

# DOCKET NO. 50-382

# WATERFORD STEAM ELECTRIC STATION, UNIT 3

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82 License No. NPF-38

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated April 24, 1992, as supplemented by letter dated February 25, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:
  - (2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 82, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Terence L. Chan, Acting Director Project Directorate IV-1

Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 8, 1993

# TO FACILITY OPERATING LICENSE NO. NPF-38

# **DOCKET NO. 50-382**

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE PAGE

INSERT PAGE

6-16

6-16

# PROCEDURES AND PROGRAMS (Continued)

- j. OFFSITE DOSE CALCULATION MANUAL implementation.
- k. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.
- 6.8.2 Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed as specified in Specification 6.5.1.6 and shall be approved by the General Manager Plant Operations prior to implementation and reviewed periodically as set forth in administrative procedures.
- 6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:
  - a. The intent of the original procedure is not altered;
  - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected;
  - c. The change is documented, reviewed by the PORC and approved by the General Manager Plant Operations within 14 days of implementation.
- 6.8.4 The following programs shall be established, implemented, and maintained:
  - a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the containment spray, safety injection, hydrogen analyzer, and the post-accident sampling system. The program shall include the following:

- Preventive maintenance and periodic visual inspection requirements, and
- 2. Integrated leak test requirements for each system at refueling cycle intervals or less.
- b. <u>In-Plant Radiation Monitoring</u>

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

# PROCEDURES AND PROGRAMS (Continued)

- 1. Training of personnel,
- 2. Procedures for monitoring, and
- 3. Provisions for maintenance of sampling and analysis equipment.

# c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- 1. Identification of a sampling schedule for the critical variables and control points for these variables,
- 2. Identification of the procedures used to measure the values of the critical variables,
- Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,
- 4. Procedures for the recording and management of data,
- 5. Procedures defining corrective actions for all off-control point chemistry conditions, and
- 6. A procedure identifying (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

## d. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1. Training of personnel,
- 2. Procedures for sampling and analysis, and
- 3. Provisions for maintenance of sampling and analysis equipment.

#### e. Basemat Monitoring

A program for monitoring of the Nuclear Plant Island Structure (NPIS) Common Foundation Basemat to ensure the continued integrity of the Basemat. The program shall include:

- settlement of the basemat
- 2. changes in ground water chemistry that could effect corrosion of reinforcing steel
- 3. seasonal variation in ground water levels
- 4. monitoring of significant cracking in the basemat.



# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

## SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 82 TO

FACILITY OPERATING LICENSE NO. NPF-38

ENTERGY OPERATIONS, INC.

WATERFORD STEAM ELECTRIC STATION, UNIT 3

**DOCKET NO. 50-382** 

## 1.0 INTRODUCTION

By application dated April 24, 1992, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Waterford Steam Electric Station, Unit 3, Technical Specifications (TS). The requested changes would remove the requirement to conduct wall-cracking surveillance as part of the basemat cracking program. By letter dated February 25, 1993, the licensee requested that the term "mapping" in the requirement statement be replaced with the term "monitoring" to more accurately describe the licensee's actions.

The February 25, 1993, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

# 2.0 **EVALUATION**

We have reviewed Waterford 3's Basemat Surveillance Program Plan, which was submitted by Louisiana Power and Light (LP&L) on June 26, 1987, and approved by the NRC staff on October 27, 1987. Section 4.c of the plan, "Wall Crack Surveillance," states that the photographic survey will continue for two additional 18-month cycles and then be discontinued if no significant changes are noted during the two surveys. By letter dated January 26, 1990, LP&L informed the NRC staff that the photographic survey had been discontinued because no significant changes had been noted during previous surveys. We have reviewed the Basemat Monitoring Program Report No. 4, dated January 1993, and found that the crack width variations in the basemat were below the allowable 15 mils. That the crack width variations in the basemat are low indicates that the crack width variations in the walls resulting from potential basemat problems should also be low, so discontinuance of photographic surveys on wall cracks is justified.

The crack surveillance for the basemat has been performed by measuring crack width at 15 locations on the basemat. Therefore, changing the word "mapping" to "monitoring" is appropriate and acceptable.

# 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comments.

# 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (57 FR 28200). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

# 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Ma

Date: July 8, 1993