

February 7, 1989

Docket No. 50-382

Mr. J. G. Dewease
Senior Vice President - Nuclear Operations
Louisiana Power and Light Company
317 Baronne Street, Mail Unit 17
New Orleans, Louisiana 70112

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Dear Mr. Dewease:

SUBJECT: ISSUANCE OF AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE
NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3
(TAC NO. 69555)

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated September 20, 1988 and November 28, 1988.

The amendment changes the Appendix A Technical Specifications and License Condition 2.C.9 by removing certain fire protection requirements in response to Generic Letters 86-10 and 88-12.

A copy of the Safety Evaluation supporting the amendment is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

David L. Wigginton, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to NPF-38
2. Safety Evaluation

cc w/enclosures:
See next page

LTR NAME: WATERFORD 3 AMEND 1/9

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
February 7, 1989

Docket No. 50-382

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Senior Vice President - Nuclear Operations
Louisiana Power and Light Company
317 Baronne Street, Mail Unit 17
New Orleans, Louisiana 70112

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Sincerely,

A handwritten signature in cursive script, appearing to read "D. L. Wigginton".

David L. Wigginton, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to NPF-38
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Jerrold G. Dewease
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Waterford 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

LOUISIANA POWER AND LIGHT COMPANY

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Louisiana Power and Light Company (the licensee) dated September 20, 1988 and November 28, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to License Condition 2.C.9 by deletion of items a. thru g. and incorporation of the following:

9. Fire Protection (Section 9.5.1, SSER 8)

The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility through Amendment 36 and as approved in the SER through Supplement 9, subject to the following provisions:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. In addition, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 50, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

4. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth L. Heston for

Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Charges to the Technical
Specifications

Date of Issuance: February 7, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 50
TO FACILITY OPERATING LICENSE NO. NPF-38
DOCKET NO. 50-382

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

3/4 3-49 thru 3/4 3-53
(5 pages)

3/4 7-29 thru 3/4 7-42
(14 pages)

B3/4 3-3a

B3/4 7-7

B3/4 7-8

6-2

6-2a

6-9

Insert

3/4 3-49 thru 53
(1 page)

3/4 7-29 thru 42
(1 page)

B3/4 3-3a

B3/4 7-7

B3/4 7-8

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Pages 3/4 7-29 through 3/4 7-42 have been deleted.

INSTRUMENTATION

BASES

for the system is thus based on testing and operating experience, and the setpoint is set at the lowest achievable IDLH gas concentration providing reliable operation and the optimum detection of toxic gases. The setpoint is therefore subject to change wherein necessitated by operating experience such as a result of changes in the Waterford 3 area chemical atmospheric profile. The setpoint is established and controlled by procedure.

3/4.3.3.8 This section deleted

3/4.3.3.9 LOOSE-PART DETECTION INSTRUMENTATION

The OPERABILITY of the loose-part detection instrumentation ensures that sufficient capability is available to detect loose metallic parts in the primary system and avoid or mitigate damage to primary system components. The allowable out-of-service times and Surveillance Requirements are consistent with the recommendations of Regulatory Guide 1.133, "Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors," May 1981.

PLANT SYSTEMS

BASES

3/4.7.9 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values.

Sealed sources are classified into three groups according to their use, with Surveillance Requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism (i.e. sealed sources within radiation monitoring or boron measuring devices) are considered to be stored and need not be tested unless they are removed from the shield mechanism.

3/4.7.10 This section deleted.

3/4.7.11 This section deleted.

3/4.7.12 ESSENTIAL SERVICES CHILLED WATER SYSTEM

The OPERABILITY of the essential services chilled water system ensures that sufficient chilled water is supplied to those air handling systems which cool spaces containing equipment required for safety-related operations and, during normal plant operation, the nonessential spaces.

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Supervisor, or during his absence from the control room, a designated individual shall be responsible for the control room command function. A management directive to this effect, signed by the Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR.
- b. The Senior Vice President - Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- c. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 UNIT STAFF

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

ADMINISTRATIVE CONTROLS

UNIT STAFF (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room.
- c. A Health Physics Technician* shall be on site when fuel is in the reactor;
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- e. Administrative procedures shall be developed and implemented to limit the working hours of individuals of the nuclear power plant operating staff who are responsible for manipulating plant controls or for adjusting on-line systems and equipment affecting plant safety which would have an immediate impact on public health and safety.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, the following guidelines shall be followed:

1. An individual shall not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual shall not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
3. A break of at least 8 hours shall be allowed between work periods, including shift turnover time.

*The Health Physics Technician and fire brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

UNIT STAFF (Continued)

4. Except during extended shutdown periods, the use of overtime shall be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager, the assistant Plant Managers, the Operations Superintendent or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime will be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- f. The Operations Superintendent shall hold a senior reactor operator license.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- g. Review of unit operations to detect potential hazards to nuclear safety.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant Manager or the Safety Review Committee.
- i. Review of the Security Plan and implementing procedures and submittal of recommended changes to the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Safety Review Committee.
- k. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PORC meeting.
- l. Review of proposed modifications to the CPC addressable constants based on information obtained through the Plant Computer-CPC data link.
- m. Review of any accidental, unplanned or uncontrolled radioactive release including reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear and to the Safety Review Committee.
- n. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL, and major changes to radwaste treatment systems.
- o. Review of the Fire Protection Program and implementation procedures and submittal of recommended changes to the Safety Review Committee.

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend in writing to the Plant Manager, prior to implementation except as provided in Specification 6.8.3, approval or disapproval of items considered under Specification 6.5.1.6a. through d. and l.
- b. Render determinations in writing, prior to implementation except as provided in Specification 6.8.3, with regard to whether or not each item considered under Specification 6.5.1.6a. through e. constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear and the Safety Review Committee of disagreements between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.1.8 The PORC shall maintain written minutes of each PORC meeting that, at a minimum, document the results of all PORC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Vice President-Nuclear and the Safety Review Committee.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering and
- h. Quality assurance practices.

COMPOSITION

6.5.2.2 The SRC shall be composed of at least five members, including the Chairman. Members of the SRC may be from within the LP&L organization or from organizations external to LP&L.

The qualifications of members selected for the SRC shall be in accordance with Section 4.7 of ANSI/ANS 3.1-1978.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 50 TO

FACILITY OPERATING LICENSE NO. NPF-38

LOUISIANA POWER AND LIGHT COMPANY

WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By letter dated September 20, 1988 and November 28, 1988, Louisiana Power and Light Company (the licensee) proposed that the existing license conditions on fire protection be replaced with the standard condition noted in Generic Letter 86-10 and also proposed changes to the Appendix A Technical Specifications (TS) for Waterford Steam Electric Station, Unit 3. The proposed changes would remove requirements for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements as recommended by Generic Letter 86-10. The proposed changes would also modify the administrative control requirements of the TS to add requirements for the Fire Protection Program that are similar to requirements for other programs implemented by license condition. Guidance on these proposed changes to TS was provided to all power reactor licensees and applicants by Generic Letter 88-12, dated August 2, 1988.

2.0 BACKGROUND

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the commission undertook a number of actions to ensure the improvements were implemented in the Fire Protection Programs for all power reactor facilities. Because of the extensive modification of Fire Protection Programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, licensees were requested by Generic Letter 86-10 to incorporate the final NRC-approved Fire Protection Program in their Final Safety Analysis Reports (FSARs). In this manner, the Fire Protection Program -- including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection -- would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the Fire Protection Program as described in the FSAR, should be used to ensure uniform enforcement of fire protection requirements. Finally, the Commission stated that with the requested actions, licensee may request an amendment to delete the fire protection TS that would now be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for license amendment requests to remove fire protection requirements from TS.

Additionally, in the licensing review of new plants, the staff has approved applicant requests to remove fire protection requirements from TS issued with the operating license. Thus, on the basis of the lead-plant proposals and the staff's experience with TS for new licenses, Generic Letter 88-12 was issued to provide guidance on removing fire protection requirements from TS.

3.0 EVALUATION

Generic Letter 86-10 recommended the removal of fire protection requirements from the TS. Although a comprehensive Fire Protection Program is essential to plant safety, the basis for this recommendation is that many details of this program that are currently addressed in TS can be modified without affecting nuclear safety. Such modifications can be made provided that there are suitable administrative controls over these changes. These details, that are presently included in TS and which are removed by this amendment, do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, do not warrant being included in TS. At the same time, suitable administrative controls ensure that there will be careful review and analysis by competent individuals of any changes in the Fire Protection Program including those technical and administrative requirements removed from the TS to ensure that nuclear safety is not adversely affected. These controls include: (1) the TS administrative controls that are applicable to the Fire Protection Program; (2) the license condition on implementation of, and subsequent changes to, the Fire Protection Program; and (3) the 10 CFR 50.59 criteria for evaluating changes to the Fire Protection Program as described in the FSAR.

The specific details relating to fire protection requirements removed from TS by the amendment include those specifications for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements. The administrative control requirements have been modified to include Fire Protection Program implementation as an element for which written procedures must be established, implemented, and maintained. In addition, the audit responsibilities of the Plant Operations Review Committee were expanded to include the review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Safety Review Committee.

The TS changes proposed by the licensee are in accordance with the guidance provided by Generic Letter 88-12, as addressed in the items below.

- (1) Specification 6.5.1.6, Unit Review Group Responsibilities, was revised to add the review of the fire protection program implementation and the submittal of recommended changes to the Safety Review Group.
- (2) Specification 3.3.3.8, Fire Detection Instrumentation, its associated Surveillance Requirements, and Bases were removed.
- (3) Specifications 3.7.1 through 3.7.1.5, Fire Suppression Systems, their associated Surveillance Requirements, and Bases were removed.
- (4) Specification 3.7.11, Fire Rated Assemblies, its associated Surveillance Requirements, and Bases were removed.
- (5) Specification 6.2.2.e on fire brigade staffing requirements was removed.

As required by Generic Letter 86-10, the licensee confirmed that the NRC-approved Fire Protection Program has been incorporated into the FSAR. Also, the licensee has proposed that the existing licensing condition on the Fire Protection Program be replaced with the standard condition noted in Generic Letter 86-10.

The licensee confirmed that the operational conditions, remedial actions, and test requirements associated with the removed fire protection TS have been included in the Fire Protection Program incorporated into the FSAR by reference to operations procedure UNT-5-013. This achieves the intent of Generic Letter 88-12 in that the procedure will be considered part of the FSAR by direct reference.

In our review of the procedure, UNT-5-013, we determined that certain portions of the TS being removed from the license were not exactly incorporated in the procedure. In discussions with the licensee, the references to TS 3.0.3 and 3.0.4 were omitted. It was suggested, but not required, that the licensee add a note on non-applicability of these sections to avoid any confusion in the future. Section 4.3.3.8.3 surveillance was dropped since the plant does not have non-supervised circuits associated with detector alarms between instruments and control room.

Under fire suppression water systems, the licensee had incorrectly dropped reference to the west fire water tanks; it will be added on Revision 1 to the procedure. In the section on spray and sprinkler systems, the licensee had changed an instruction which could result in uncertainty by technicians performing surveillances. As suggested to the licensee and as agreed to, the note on page 27 of the procedure will be revised to read "The charcoal filter unit spray nozzles need only be visually inspected for obstructions each time the charcoal is removed." A fire detection instrument has been added to the table on page 37 in zone FSB4 in the Fuel Handling Building; this is an oversight by the licensee in the original TS. Several other

typographical errors were noted to the licensee. Our review of these changes resulted in licensee's commitment to correct the procedure in Revision 1. We find this and the procedure acceptable.

4.0 CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Administrator, Nuclear Energy Division, Office of Environmental Affairs, State of Louisiana of the proposed determination of no significant hazards consideration. No comments were received.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

Based upon its evaluation of the proposed changes to the Waterford 3 license condition and Technical Specifications, the staff has concluded that: there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. The staff, therefore, concludes that the proposed changes are acceptable, and are hereby incorporated into the Waterford 3 Technical Specifications.

Dated: February 7, 1989

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