**GE** Nuclear Energy

# **ESBWR Design, Technology and Program Plan Overview**

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## **Purpose of Meeting**

#### Provide ESBWR background information to NRC staff

- Issues to be resolved during pre-application review
  - Adequacy of testing, analysis methodology approval, SSAR details
- Reference design
- Testing and technology basis
- Analysis methodology, qualification and application approach
- Information on submittals to be made in August/Sept 2002

#### Obtain NRC feedback

- Overall approach
- Identification of additional information needed by NRC for completing pre-application review
- Schedule and steps for reaching agreement on pre-application review scope, schedule and cost and overall plan

## Agenda for ESBWR Meetings June 20/21, 2002

#### Introduction and overview

- Pre-application plan
- Design description
  - Passive safety systems
- Plant performance
- Technology basis
  - Testing and analysis program
  - Testing program overview
  - Scaling
  - Methodology, qualification and application

#### Summary and Conclusion

# **Enhanced performance and economics and a solid technology basis provide confidence in the design**

## **Outline – Introduction and Overview**

- History of ESBWR development
- Design summary
- Passive safety systems
- Plant performance
- Technology basis
- Pre-application plan
- Summary and Conclusion

# 12 month plan to obtain cost and schedule for ESBWR certification



**Evolution of the BWR** 

# **ESBWR** History – showing inter-relationship to SBWR program (a program to enhance plant performance and economics)



## **Outline**

History of ESBWR development
Design summary
Passive safety systems
Plant performance
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# **Evolution of the ESBWR Reactor Design**



## **Evolution and Innovation Towards Simplicity**

## **Evolution of BWR Containments**



**Evolution and Innovation Towards Simplicity** 

## **Comparison of Key Parameters**

<u>Parameter</u>	BWR/4-Mk I (Browns Ferry 3)	BWR/6-Mk III (Grand Gulf)	<u>ABWR</u>	<u>ESBWR</u>
Power (MWt)	3293	3833	3926	4000
Power (MWe)	1098	1290	1350	1380
Vessel height (m)	21.9	21.8	21.1	27.7
Vessel diameter (m)	6.4	6.4	7.1	7.1
Fuel Bundles (number)	764	800	872	1020
Active Fuel Height (m)	3.7	3.7	3.7	3.0
Power density (kw/l)	50	54.2	51	54
Number of CRDs	185	193	205	121

## **Evolution towards simplicity within a small range**



## **Natural Circulation**

#### Reduction in Components

- pumps, controls, power supplies
- vessel internals
- control rod drives

#### • Very Good Performance and Reliability

- power/flow ratio same as pumped plant at rated power
- large margin to stability

## Simplified systems improve plant reliability

#### **ESBWR Plant Schematic**



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# **Passive Safety Systems**

#### Reactivity Control

- Accumulator driven scram system
- Accumulator driven backup boron injection system

#### Inventory Control

- Taller vessel with additional inventory
- High pressure isolation condensers (IC)
- Depressurization and gravity driven cooling system (GDCS)

#### Decay Heat Removal

- Isolation condensers for transients and station blackout
- Pressure suppression system for blowdown energy
- Passive Containment Cooling System (PCCS) for breaks
  - Decay heat removal capability for 72 hours without operator action
  - Simple actions extend capability beyond 72 hours
  - Plant recovery actions expected in 24 hours
  - Containment overpressure system (COPS) defense in depth

#### **Simplified Systems Using Operating Plant Technology**

### Basic safety system design criteria – maintain water inventory

#### Increase inventory in the vessel

- Use taller vessel
- Increase amount of subcooled water
- Minimize inventory loss from the vessel
  - Eliminate large pipes below the core and minimize other pipes
  - Use saturated systems with typical BWR pressure for normal operation
- Keep core covered after initial blowdown
  - Shorter core lower in the vessel

#### Provide Inventory Makeup

- Required makeup rate is very low
  - Multiple tanks rely on gravity
- No high capacity systems needed
- Fewer systems result in minimal system interaction

#### Gravity Driven Cooling System (GDCS) - Main Steam Line Break





# Safety Systems Inside Containment Envelope



#### **Decay Heat Removal**

From the vessel - no pipe break

#### From the containment

- no break
- small leak
- large break
- severe accident with hydrogen

#### **Concept is simple and evaluation is straightforward**

#### **Decay Heat Removal Systems**

#### Remove Decay Heat From Vessel

- Main Condenser
- Normal shutdown cooling system
- Isolation condensers
- Remove vessel heat through relief valve opening

#### • If Needed, Remove Heat From Containment

- Suppression pool cooling (non-safety)
- Passive containment cooling (PCC) Hx (safety-grade)
  - Always available
  - Drywell/wetwell pressure difference drives the steam and noncondensable flow through the heat exchangers
  - Condensed steam returns to drywell/vessel, non-condensables collect in the wetwell airspace

#### **Several Diverse Means of Decay Heat Removal**

#### Decay Heat Removal from Vessel – How it Works

- Isolation Condenser simple tube heat exchanger in a pool of water
- Redundant, fail-safe valves open and water drains from tubes
- Steam flows naturally to tubes and is condensed and condensate returns to vessel, energy transferred to pool outside containment



## Plant Features that Result in Optimum IC Response

Large Reactor Vessel

- slow pressure rate
- no need for fast action
- no loss of water inventory following isolation

 Heat Sink Pool Outside Containment

- no containment heat up
- very easy to extend period of cooling

# Isolation condenser is a proven system that has been used in many operating plants

## **Decay Heat Removal from Containment - How it works**

- Initially steam (blowdown energy) flows to large heat sink in containment (suppression pool) and through heat exchangers
- Longer term (decay heat) steam flows to heat exchanger and heat is transferred outside containment
  - Vertical tube heat exchangers in a pool of water

Concept is simple, reliable - extensive testing and analysis provide high confidence in the design margin



#### Plant Features Resulting in Optimum Containment Response

Pressure suppression system

- Compact heat exchangers
- Heat sink pool outside containment

- no need for fast action
- allows heat exchanger to operate in forced flow conditions
- allow substantial margin in design without economic penalty
- very easy to extend period of cooling

## Containment heat exchangers are a major simplification Modular design allows easy scale-up in power

### **Comparison of Safety System - Passive vs. Active**



#### SBWR (670 MWe)

#### ESBWR (1380 MWe)



#### Significant Reduction in Systems & Buildings – scale up with innovations

#### **ELEVATION COMPARISONS**





#### ABWR

## ESBWR

Significant Reduction in Systems & Buildings with passive systems

## PLAN VIEW AT GRADE LEVEL



ABWR



Significant Reduction in Systems & Buildings with passive systems

## **Outline**



## Plant performance margins enhanced by design changes

## **Comparison of Natural Circulation Flow for BWRs**



## **ESBWR** has large margin to unstable regions

#### Reactor pressure response to isolation events



#### **ESBWR** has slower pressurization - no relief valves open

# Water Level in Shroud Following a Typical Break

(values are intended to show typical trends for limiting breaks)



## Large margins should facilitate regulatory review

## **Containment Pressure Following a Pipe Break**

(values are intended to show typical trends for limiting breaks – ESBWR has lower design pressure than ABWR/SBWR)



## Large margin to design pressure even without COP system

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# Extensive SBWR testing supplemented with additional ESBWR tests

## **Extensive Technology Program for Features New to SBWR**

#### Component tests

- Full scale components tests DPV valves and vacuum breaker
- Full scale isolation condensers & PCCS heat exchangers,

#### Integral tests

- Integral tests at different scales 1/400 to 1/25
- System interaction tests
- Large hydrogen releases
- Testing used to qualify computer codes
- Extensive international cooperation
- Extensive review and participation by NRC staff
  - Test matrix
  - Running of actual tests

## A Complete, Multi-year International Technology Program Supports the Design

## Safety System (GIST) Test Facility and Depressurization Valve



Reactor Depressurization Valve in the Test Facility



# Prototype Vacuum Breaker



## Additional testing for ESBWR configuration

#### Decay Heat Removal

8 Integrated system tests run in PANDA - TEPSS Pre- and post-test predictions

o TRACG, TRAC-BF1, RELAP5 and MELCOR

Natural circulation high pressure stability tests at CRIEPI

#### Multi-year program extended the SBWR database to ESBWR

## **ESBWR Program Summary**

#### 🔶 9 year ESBWR program

- Reduced Components and Systems simplify
- Reduced the Structures and Buildings simplify

#### • 9 year Technology Studies

- Large margins confirmed increased over SBWR
- Qualified codes for incremental changes for ESBWR

#### Challenges for the Coming Years

 Can simple design, large margins and completed testing simplify the regulatory challenges?

Improved Performance and Economics Completed Extensive Technology Program SBWR/ABWR ease Regulatory Challenges for ESBWR

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# 12 month plan to obtain cost and schedule for ESBWR certification

## **Pre-application review plan**

12 month plan to obtain cost and schedule for certification

- Complete plant description provided as reference not for review
- ♦ 3 specific issues to be addressed
  - Adequacy of testing for passive safety systems
  - Approval of methodology for evaluating plant performance
  - Technical details needed in the SSAR

#### Key dates

- Submittals for review August 2002
  - Some supplements to be provided September 2002
- Adequacy of testing– Dec 2002
- Initial assessment on methodology Dec 2002
- Completion of pre-application review April 2003

## Based on timely submittals and responses to RAI's Address NRC issues and concerns from SBWR review

## ESBWR Pre-application Schedule



The above DRAFT plan shows the key steps to achieve the following:

- 1. Phase 1 agreement by July 2002 to define the issues to be resolved during the pre-application review. This agreement to include NRC cost and schedule for the Phase 2 activities.
- 2. Phase 2 activities are expected to cover the following 3 issues:
  - 2.1 Assessment of the technology basis for the passive safety systems
  - 2.2 Approval of the Analysis Methodology for analyzing plant transient, LOCA, containment response
  - 2.3 Definition/agreement on SSAR and certification document details
  - 2.4 Cost and schedule for certification
- 3. Phase 2 activities are scheduled based on getting early evaluation of 2.1 and 2.2 and given that the design has large margins to LOCA and containment limits.
- 4. The DRAFT schedule is based on timely GE submittals especially the ones defining the start of Phase 2 in August and timely responses to RAI's.

## **Basis for schedule**

#### Adequacy of testing issue

- NRC completed review of SBWR testing and analysis program
  - Found to be adequate RAI's covered issues that did not affect conclusion
- Additional ESBWR testing done for specific configuration changes
- Scaling report covers the test programs
  - Found to be adequate RAI's were addressed

#### Approval of analysis methodology – TRACG

- Model description and qualification report completed for operating plants
  - Supplement extending qualification to passive safety systems (SBWR)
  - Supplement covering ESBWR specific tests
- Application methodology
  - Transients same as operating plants
  - LOCA and containment bounding approach
- Large margins in plant performance based on design features
- Plant bounding response can be calculated/analyzed easily

#### **Basis for schedule (cont.)**

Design certification submittal details

Rely on previous reviews regarding details/approach

## Extensive SBWR submittals and reviews, new test data and reports, coupled with design changes to add margin, provide a solid basis

## ESBWR Design/Technology based on SBWR and ABWR



## **Summary and Conclusions**

#### Passive safety systems have simplified the plant design

- Plant evaluations are simpler and rely on less complex analyses
- Substantial margins exist in the design
- Defense in depth systems provide back-up

#### ESBWR is an optimized design

- Simplified the design
- Improved operation and maintenance
- Enhanced the plant economics
- Next significant step is certification
  - Design Certification schedule and resources are a key issue

## **Enhanced performance and economics and a solid technology basis provide confidence in the design**