

Table of Accomplishments	
Activity	Accomplishment
Reactor Oversight Process (ROP)	Based on its assessment of stakeholder feedback and the results and lessons learned from annual self-assessments, the staff has developed a much greater level of confidence that the ROP has met the Commission's direction to develop an oversight process that is more objective, risk-informed, understandable, and predictable. The most recent self assessment concluded that the risk-informed ROP helps the industry and the NRC to focus resources on areas of the most safety significance. The staff has identified areas for improvement of performance indicators and of the significance determination process and has developed plans to accomplish those improvements.
Guidance for risk-informed licensing basis changes	The staff issued a Regulatory Issue Summary that advised the industry of staff guidance for applying risk-informed decision making in the review of non-risk-informed license amendment requests. The staff has also prepared and is using guidance for the review of risk-informed licensing basis changes in the areas of graded quality assurance, inservice inspection, and inservice testing. Final update of this guidance is ongoing, but is dependent on progress in other areas, particularly special treatment requirements as discussed below.
Special Treatment Requirements	The staff developed draft rule language, obtained stakeholder feedback (from public meetings and by written comments), and is continuing to prepare the proposed rule package. In February, the staff provided comments to NEI on the proposed implementation guidance on categorization. In March 2002 the staff completed its interactions on pilot plant conduct of integrated decision making panel reviews of candidate systems. In April, the staff prepared draft staff review guidance for review of a PRA to be used in this application which was peer-reviewed using NEI 00-02.
10 CFR Part 50.44	The staff completed a detailed technical review that provided the basis for proposed risk-informed changes to the rule. The improved realism supports the agency's decision to eliminate requirements for equipment that is not important to safety. The Commission approved the staff's planned approach. The staff completed the proposed rulemaking package in May and provided it to the Commission for approval.
PRA Quality	The staff has been working closely with ASME, ANS, NFPA, and NEI to develop standards for PRA quality and PRA review. Since the December 2001 version of the RIRIP, ASME has issued its final standard for Level 1 and limited Level 2 PRA; ANS has continued to develop standards to address external hazards, low power/shutdown, and internal fire events; and NEI has issued its guidance on Level 1 and simplified Level 2 PRA peer review. The staff has begun writing a new regulatory guide and SRP chapter to endorse the ASME standard and related guidance.
10 CFR Part 50.46	The staff has nearly completed the technical studies for each of the proposed changes to 50.46 and its associated rules. The technical reports related to ECCS evaluation criteria (Appendix K), ECCS acceptance criteria, and ECCS functional reliability (GDC 35) are expected to be completed by July 2002. A report on one aspect of the ECCS reliability study, a plant-specific approach to assessing ECCS functional reliability, was delivered in May 2002 and is currently being reviewed by a working group dedicated to drafting the alternative rule to GDC 35. The long-term study related to estimating break frequency according to pipe size is still ongoing.
Risk-Informed Technical Specifications	The staff completed reviews of industry proposals to modify requirements related to surveillance requirements and preferred end states. Seven other industry initiatives have been proposed to the staff. Safety evaluations of two initiatives are in progress. Reviews of two other initiatives are ongoing.
Individual Plant Examination - External Events (IPEEE)	The staff completed its review of the industry's IPEEE submittals and an insights report was prepared. The IPEEE program was a success that resulted in the nuclear power industry identifying safety improvements that substantially reduced the risk of accidents. The generic insights from this effort will be used to support development of PRA guidance and standards, while plant-specific risk information will support the risk-informed reactor oversight program.

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Reg. Guide 1.174/SRP Chapter 19	The staff completed its first revision of these documents since their initial publication in July 1998, with the following changes: <ol style="list-style-type: none"> 1. Risk related information may now be requested if new, unforeseen hazards emerge or prospects increase substantially for known hazards. 2. Indication was provided of on-going staff discussions on the effect of increases to fuel burn-up and changes to mixed-oxide fuel on risk metrics, such as large early release frequency. 3. Inclusion of additional examples of risk insights in the decision-making process.
NMSS Risk Case Studies	The eight case studies were completed in December 2001. As part of this effort, NMSS held several stakeholder meetings, including a meeting with a diverse set of Stakeholders in October 2001. Also, to gain a broader perspective of risk in the materials and waste arenas, the eight case studies were integrated with other related risk assessments. Major outcomes of the case study effort were: (1) development of a formal set of Screening Considerations that could be used to determine whether an NMSS regulatory activity should be risk-informed, (2) development of a guide for using the Screening Considerations, and (3) and formation of a framework for developing materials and waste safety goals.
Identification of Regulatory Activities Amenable to Increased Use of Risk-Information	Between January and April 2002, the Risk Task Group, in consultation with the NMSS Divisions, used the NMSS Screening Considerations to systematically identify NMSS regulatory activities that are amenable to being risk-informed. This identification of activities will serve as the NMSS road map towards comprehensively risk-informing its regulatory activities. Actual implementation of the activities will be planned, prioritized and budgeted through the PBPM process.
Development of Materials and Waste Safety Goals	As part of the case study effort, NMSS established the feasibility and usefulness of safety goals in the materials and waste arenas and developed a first draft of safety goals. NMSS and RES have initiated a joint effort to continue developing materials and waste safety goals and risk metrics, and to develop other tools, methods, data, guidance and standards necessary for implementing risk-informed approaches in NMSS.
NMSS Risk Training Program	NMSS has instituted training courses to advance the use of risk assessment and risk management into its day-to-day operations. Tier I and II training courses on risk assessment in NMSS and a Tier III course on quantitative frequency analyses are offered regularly. A Tier III course on byproduct materials system of risk analysis and evaluation has been developed and the pilot course will be offered in June 2002. Evaluation of Tier III training programs for risk specialists is ongoing.
NMSS Risk Communication plan	In April 2002, NMSS revised the "Communication Plan for Risk Informing Materials and Waste Regulations." The Communication Plan describes NMSS' plan for communicating risk information to internal and external stakeholders. The purpose of the plan is (1) to communicate, to external stakeholders, the major points of the program to risk inform materials regulations, in order to increase public confidence, and (2) to communicate, within the NRC, the NMSS Risk Task Group's activities, to increase understanding and acceptance of NMSS's risk-informing efforts and to assist NMSS staff in communicating risk-related information to external stakeholders.
Risk-informed, Performance-based Temporary Instruction for the Nuclear Medicine Program	The staff's medical pilot program (nuclear medicine program) to streamline inspection and enforcement of materials licensees was completed in January 2002 with the results of the pilot program reported to the commission. A risk-informed, performance-based Temporary Instruction (TI) for the medical pilot program used a focus element approach to assess a licensee's performance relative to desired safety-related outcomes. The approach is expected to reduce unnecessary regulatory burden through more efficient and effective inspections. The pilot was successful in reducing direct inspection hours and making the inspections more consistent. Because of the success, the temporary instruction was extended until the inspection procedures are changed to incorporate all of the Materials Phase II changes.

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Part 70 Integrated Safety Analyses	In accordance with the revised 10 CFR Part 70, each licensee has submitted a plan for conducting its Integrated Safety Analysis (ISA) for NRC staff review. The NRC staff has reviewed and approved two of the six plans. The NRC staff anticipates approving the remaining four plans during the summer of 2002. With much stakeholder involvement, the 10 CFR Part 70 Standard Review Plan was finalized in December 2001 and published in March 2002.
Fuel Cycle Oversight Revision Project	In March 2002, the staff provided the Commission a status report on the fuel cycle oversight revision project. This project will be closed at the end of FY 2002, after the staff completes near-term revisions of the Licensee Performance Review process and the guidance for conducting the fuel cycle facility inspection program. Beginning in FY 2003, risk-informed revisions to the fuel cycle oversight program's inspection procedures will be made during normal updates of the inspection program, commensurate with the implementation of the Part 70 revisions. Under this approach, the fuel cycle facility oversight process will evolve in a more risk-informed direction over the next several years.
10 CFR Part 63	The staff published the final risk-informed, performance-based rule for disposal of high-level radioactive wastes in the proposed geologic repository at Yucca Mountain, Nevada.
10 CFR Part 63 Guidance	The staff published in March 2002 the NUREG-1804, Revision 2, "Yucca Mountain Review Plan (Draft Report for Comment)." The review plan provides guidance to staff on implementing the risk-informed, performance-based regulations of Part 63.
Decommissioning Guidance Consolidation	The Decommissioning Guidance Consolidation Project is reviewing and consolidating existing decommissioning guidance, updating and risk-informing the guidance, as appropriate, in the process. Staff held a public workshop in June 2001 to solicit feedback from the public and stakeholders on the project. Also, staff convened the Volume 1 writing team in June 2001. The staff published Volume 1 (Decommissioning Process) as NUREG-1757 for comment in January 2002. Volume 1 is expected to be issued in September 2002. The writing team for Volume 2 (Dose modeling) was convened in January 2002 and the draft will be issued for comment in September 2002.