

February 9, 1988

Docket No. 50-382

Mr. J. G. Dewease
Senior Vice President - Nuclear Operations
Louisiana Power and Light Company
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Dear Mr. Dewease:

SUBJECT: ISSUANCE OF AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE
NPF-38 - WATERFORD STEAM ELECTRIC STATION, UNIT 3
(TAC NO. 66294)

The Commission has issued the enclosed Amendment No. to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 28, 1987.

The amendment changes the Appendix A Technical Specifications by adding Limiting Conditions for Operation for as-built primary and backup overcurrent protection devices used to protect the polar crane's electrical penetration.

A copy of the Safety Evaluation supporting the amendment is also enclosed. Notice of Issuance will be included in the Commission's next Bi-weekly Federal Register notice.

Sincerely,

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James H. Wilson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. to NPF-38
2. Safety Evaluation

cc w/enclosures:

See next page

LTR NAME: WATERFORD 3 AMENDMENT NO. 67

PD4/LA
PNoonan
12/21/87

PD4/P
JWilson:
12/29/87

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SELB
F
12/1/87
1/4/88

OGC-Bethesda
1/20/88

PD4/D
JCalvo
2/19/88

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Mr. Jerrold G. Dewease
Louisiana Power & Light Company

Waterford 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

LOUISIANA POWER AND LIGHT COMPANY

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Louisiana Power and Light Company (the licensee) dated August 28, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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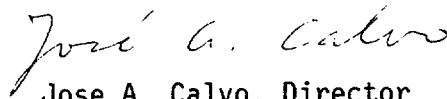
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 29, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 9, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 29
TO FACILITY OPERATING LICENSE NO. NPF-38
DOCKET NO. 50-382

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove

3/4 8-19

Insert

3/4 8-19

TABLE 3.8-1 (Continued)
480 VOLTS POWER FROM LOW VOLTAGE SWITCHGEAR

ITEM NO.	SYSTEM POWERED	PROTECTION	OVERCURRENT PROTECTIVE DEVICES (NOTE 6)		TIME-CURRENT CHARACTERISTIC	REMARKS
			TYPE AND TRIP SETPOINT (NOTE 1)	LINE NO.		
			SHEET NO.			
1	Polar Crane	Primary Backup	20A1,20A2	16, 16	Notes 2, 3	Item 1 - The Backup Protection is accomplished by Transfer Trip Relays similar to the illustration shown on FSAR Figure 8.3-29. However, when polar crane is not required for use, Primary Breaker is to be Locked Out in the Open Position during Modes 1, 2, 3 and 4.
2	CEDM Cooling Unit E-16(3A)	Primary (Note 4) Backup	20A1,20A2	17, 17	Notes 2, 3	Items 2 through 5 - The Backup Protection is accomplished by Transfer Trip Relays as illustrated on FSAR Figure 8.3-29.
3	CEDM Cooling Unit E-16(3C)	Primary (Note 4) Backup	20A3,20A4	23, 23	Notes 2, 3	Adjust Transfer Trip Relay 2/1140 to 1 s.
4	CEDM Cooling Unit E-16(3D)	Primary (Note 4) Backup	21A3,21A4	24, 24	Notes 2, 3	Adjust Transfer Trip Relay 2/1142 to 1 s.
5	CEDM Cooling Unit E-16(3B)	Primary (Note 4) Backup	21A1,21A2	18, 18	Notes 2, 3	Adjust Transfer Trip Relay 2/1141 to 1 s.

TABLE 3.8-1 (Continued)
480 VOLTS POWER FROM LOW VOLTAGE SWITCHGEAR (Continued)

ITEM NO.	SYSTEM POWERED	PROTECTION	OVERCURRENT PROTECTIVE DEVICES (NOTE 6)		TIME-CURRENT CHARACTERISTIC	REMARKS
			TYPE AND TRIP SETPOINT (NOTE 1) SHEET NO.	LINE NO.		
6	Press. Heaters Backup Bank 1 (B-1)	Primary (Note 4) Backup	23A1,23A2 Adjust Transfer Trip Relay 2/285 to 0.5 s.	4, 4	Notes 2, 5	Items 6 through 11 - The Backup Protection consists of Transfer Trip Relays activated by any one of the Primary Overcurrent Protective Relays as illustrated on FSAR Figure 8.3-30.
7	Press. Heaters Backup Bank 2 (B-2)	Primary (Note 4) Backup	23A1,23A2 Adjust Transfer Trip Relay 2/286 to 0.5 s.	5, 5	Notes 2, 5	
8	Press. Heaters Backup Bank 3 (B-3)	Primary (Note 4) Backup	23A1,23A2 Adjust Transfer Trip Relay 2/287 to 0.5 s.	6, 6	Notes 2, 5	
9	Press. Heaters Backup Bank 4 (B-4)	Primary (Note 4) Backup	24A1,24A2 Adjust Transfer Trip Relay 2/288 to 0.5 s.	4, 4	Notes 2, 5	
10	Press. Heaters Backup Bank 5 (B-5)	Primary (Note 4) Backup	24A1,24A2 Adjust Transfer Trip Relay 2/289 to 0.5 s.	5, 5	Notes 2, 5	
11	Press. Heaters Backup Bank 6 (B-6)	Primary (Note 4) Backup	24A1,24A2 Adjust transfer Trip Relay 2/290 to 0.5 s.	6, 6	Notes 2, 5	



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 29 TO

FACILITY OPERATING LICENSE NO. NPF-38

LOUISIANA POWER AND LIGHT COMPANY

WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By application dated August 28, 1987, Louisiana Power & Light Company (LP&L or the licensee) requested changes to the Technical Specifications (Attachment A to Facility Operating License No. NPF-38) for Waterford Steam Electric Station, Unit 3. The proposed changes would add Limiting Conditions for Operation for as-built primary and backup overcurrent protective devices used to protect the polar crane's containment electrical penetration to Technical Specification 3.8-1.

2.0 DISCUSSION

Presently, Technical Specification Limiting Condition for Operation (LCO) 3.8.4.1 requires all containment penetration conductor overcurrent protection devices shown on Table 3.8-1 to be operable whenever the reactor is in Modes 1, 2, 3, or 4. With respect to the polar crane, Table 3.8-1 currently states that the primary breaker is locked out in the open position during operation in Modes 1, 2, 3, and 4; hence there is no requirement to list the actual primary or backup protection. The proposed changes will remove the sentence stating that the primary breaker is locked out in the open position during operation in Modes 1, 2, 3, and 4 and insert the as-built overcurrent protective devices that are actually installed.

3.0 EVALUATION

As stated in the basis for Technical Specification 3.8.4.1 the requirement for overcurrent protection is to ensure "Containment electrical penetrations and penetration conductors are protected by either deenergizing circuits not required during reactor operation or by demonstrating the operability of primary and backup overcurrent protection circuit breakers during periodic surveillance". Presently, for the polar cranes, the licensee takes the former action to ensure adequate protection. However, adequate overcurrent protection devices are part of the as-built configuration of the polar crane electrical system. The proposed changes would impose surveillance requirements similar to those for other electrical penetrations, allowing the licensee to use the polar crane in Modes 1 through 4. The proposed changes, therefore, remain within the bounds of the bases of the Technical Specification.

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In addition, when using the polar crane the potential exists for dropping heavy loads which could impact irradiated fuel in the reactor vessel or equipment necessary for safe shutdown of the reactor. As a result the NRC in July 1980 published NUREG-0612 "Control of Heavy Loads at Nuclear Power Plants" in addition to the issuance of a Generic Letter in December 1980. In its response, the licensee stated its requirements for safe load paths, crane operator qualifications, crane inspection and testing. A review of these submittals indicated that the licensee satisfied all applicable requirements of NUREG-0612. The proposed changes will not affect the requirements stated in the licensee's submittals.

Therefore, for the reasons stated above, the staff concludes that the proposed changes to Table 3.8-1 of the Technical Specifications pose no safety concern.

4.0 CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Administrator, Nuclear Energy Division, Office of Environmental Affairs, State of Louisiana of the proposed determination of no significant hazards consideration. No comments were received.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in installation or use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

Based upon its evaluation of the proposed changes to the Waterford 3 Technical Specifications, the staff has concluded that: there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. The staff, therefore, concludes that the proposed changes are acceptable, and are hereby incorporated into the Waterford 3 Technical Specifications.

Dated: February 9, 1988

Principal Contributor: J. Wilson