



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
March 4, 1987

Docket No.: 50-382

Mr. J. G. Dewease  
Senior Vice President - Nuclear Operations  
Louisiana Power and Light Company  
317 Baronne Street, Mail Unit 17  
New Orleans, Louisiana 70160

Dear Mr. Dewease:

SUBJECT: ISSUANCE OF AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-38  
FOR WATERFORD 3

The Commission has issued the enclosed Amendment No. 17 to Facility Operating License No. NPF-38 for the Waterford Steam Electric Station, Unit 3. The amendment consists of changes to the Technical Specifications in response to your applications transmitted by letters dated August 20 and October 1, 1986.

The amendment changes the Appendix A Technical Specifications by: revising the reporting requirements for the release of liquid radioactive effluents to unrestricted areas; revising the action to be taken in the event that the radioactive liquid effluent monitoring instrumentation is inoperable, and revising the action to be taken in the event that the radioactive gaseous effluent monitoring instrumentation is inoperable.

A copy of the Safety Evaluation supporting the amendment is also enclosed.

Sincerely,

A handwritten signature in cursive script that reads "James H. Wilson".

James H. Wilson, Project Manager  
PWR Project Directorate No. 7  
Division of PWR Licensing-B

Enclosures:

1. Amendment No. 17 to NPF-38
2. Safety Evaluation

cc: See next page

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Waterford 3

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ISSUANCE OF AMENDMENT NO. 17 TO FACILITY OPERATING  
LICENSE NO. NPF-38 FOR WATERFORD 3

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Docket File 50-382

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

LOUISIANA POWER AND LIGHT COMPANY

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 17  
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment, dated August 20 and October 1, 1986 by Louisiana Power and Light Company (licensee), comply with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

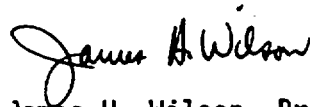
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 17, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in this license. LP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James H. Wilson, Project Manager  
PWR Project Directorate No. 7  
Division of PWR Licensing-B

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 4, 1987

- 3 -

ATTACHMENT TO LICENSE AMENDMENT NO. 17  
TO FACILITY OPERATING LICENSE NO. NPF-38  
DOCKET NO. 50-382

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

<u>Amendment Pages</u>	<u>Overleaf Pages</u>
3/4 3-55	3/4 3-56
3/4 3-57	3/4 3-58
3/4 3-60	3/4 3-59
3/4 3-63	-
3/4 3-64	-
3/4 11-1	3/4 11-2

## INSTRUMENTATION

### RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

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3.3.3.10 The radioactive liquid effluent monitoring instrumentation channels shown in Table 3.3-12 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Specification 3.11.1.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined and adjusted in accordance with the methodology and parameters in the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: At all times.

ACTION:

- a. With radioactive liquid effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above Specification, immediately suspend the release of radioactive liquid effluents monitored by the affected channel, or declare the channel inoperable.
- b. With less than the minimum number of radioactive liquid effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3-12. Restore the inoperable instrumentation to OPERABLE status within 30 days or, if unsuccessful, explain in the next Semiannual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.8, why this inoperability was not corrected within the time specified. Releases need not be terminated after 30 days provided the specified ACTIONS are continued.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.3.10 Each radioactive liquid effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3-8.



TABLE 3.3-12

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
1. RADIOACTIVITY MONITORS PROVIDING ALARM AND AUTOMATIC TERMINATION OF RELEASE		
a. Boric Acid Condensate Discharge	1	28
b. Waste, Waste Condensate and Laundry Discharge	1	28
c. Dry Cooling Tower Sumps	1/sump	29
d. Turbine Building Industrial Waste Sump	1	29
e. Circulating Water Discharge (Blowdown Heat Exchanger and Auxiliary Component Cooling Water Pumps) <sup>#</sup>	1	29
2. CONTINUOUS COMPOSITE SAMPLERS		
a. Steam Generator Blowdown Effluent Line	1	29
3. FLOW RATE MEASUREMENT DEVICES		
a. Boric Acid Condensate Discharge	1	30
b. Waste, Waste Condensate and Laundry Discharge	1	30
c. Turbine Building Industrial Waste Sump*	N.A.	N.A.
d. Dry Cooling Tower Sumps*	N.A.	N.A.
e. Circulating Water Discharge* (Blowdown and Blowdown Heat Exchanger and Auxiliary Component Cooling Water Pumps)	N.A.	N.A.

<sup>#</sup>Automatic termination of blowdown discharge only

TABLE 3.3-12 (Continued)

TABLE NOTATIONS

\*Pump performance curves generated in place shall be used to estimate flow.

ACTION STATEMENTS

- ACTION 28 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided best efforts are made to repair the instrument and that prior to initiating a release:
- a. At least two independent samples are analyzed in accordance with Specification 4.11.1.1.1, and
  - b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge line valving;
- ACTION 29 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided best efforts are made to repair the instrument and that grab samples are analyzed for radioactivity at a lower limit of detection of at least  $10^{-7}$  microcurie/mL.
- a. At least once per 12 hours when the specific activity of the secondary coolant is greater than 0.01 microcurie/gram DOSE EQUIVALENT I-131, or
  - b. At least once per 24 hours when the specific activity of the secondary coolant is less than or equal to 0.01 microcurie/gram DOSE EQUIVALENT I-131.
- ACTION 30 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided best efforts are made to repair the instrument and that the flow rate is estimated at least once per 4 hours during actual releases. Pump performance curves generated in place may be used to estimate flow.

TABLE 4.3-8

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. RADIOACTIVITY MONITORS PROVIDING ALARMS AND AUTOMATIC TERMINATION OF RELEASE				
a. Boric Acid Condensate Discharge	P	P	R(3)	Q(1)
b. Waste, Waste Condensate and Laundry Discharge	P	P	R(3)	Q(1)
c. Dry Cooling Tower Sumps	D	M	R(3)	Q(5)
d. Turbine Building Industrial Waste Sump	D	M	R(3)	Q(5)
e. Circulating Water Discharge (Blowdown Heat Exchanger and Auxiliary Component Cooling Water Pumps)	D	M	R(3)	Q(5)
2. CONTINUOUS COMPOSITE SAMPLERS				
a. Steam Generator Blowdown Effluent Line	D(6)	N.A.	R	Q
3. FLOW RATE MEASUREMENT DEVICES				
a. Boric Acid Condensate Discharge	D(4)	N.A.	R	Q
b. Waste, Waste Condensate and Laundry Discharge	D(4)	N.A.	R	Q
c. Turbine Building Industrial Waste Sump	N.A.	N.A.	N.A.	N.A.
d. Dry Cooling Tower Sumps	N.A.	N.A.	N.A.	N.A.
e. Circulating Water Discharge (Blowdown and Blowdown Heat Exchangers and Auxiliary Component Cooling Water Pumps)	N.A.	N.A.	N.A.	N.A.

#Automatic termination of Blowdown discharge only

TABLE 4.3-8 (Continued)

TABLE NOTATION

- (1) The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occur if any of the following conditions exists:
  1. Instrument indicates measured levels above the alarm/trip setpoint.
  2. Circuit failure.
  3. Instrument indicates a downscale failure.
- (2) The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exists:
  1. Instrument indicates measured levels above the alarm setpoint.
  2. Circuit failure.
- (3) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference standards certified by the National Bureau of Standards or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system for over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration shall be used.
- (4) CHANNEL CHECK shall consist of verifying indication of flow during periods of release. CHANNEL CHECK shall be made at least once per 24 hours on days on which continuous, periodic, or batch releases are made.
- (5) The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway occurs if the instrument indicates measured levels above the alarm/trip setpoint and that control room alarm annunciation occurs if any of the following conditions exists:
  1. Instrument indicates measured levels above the alarm set.
  2. Circuit failure.
  3. Instrument controls not set in operate mode.
- (6) CHANNEL CHECK shall be made at least once per 24 hours on days on which continuous releases are made to the Circulating Water System or Waterford 3 waste pond.

## INSTRUMENTATION

### RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

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3.3.3.11 The radioactive gaseous effluent monitoring instrumentation channels shown in Table 3.3-13 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Specification 3.11.2.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined and adjusted in accordance with the methodology and parameters in the ODCM.

APPLICABILITY: As shown in Table 3.3-13.

ACTION:

- a. With a radioactive gaseous effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above Specification, immediately suspend the release of radioactive gaseous effluents monitored by the affected channel, or declare the channel inoperable, or change the setpoint so it is acceptably conservative.
- b. With less than the minimum number of radioactive gaseous effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3-13. Restore the inoperable instrumentation to OPERABLE status within 30 days or, if unsuccessful, explain in the next Semi-annual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.8, why this inoperability was not corrected within the time specified. Releases need not be terminated after 30 days provided the specified ACTIONS are continued.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.3.11 Each radioactive gaseous effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3-9.

TABLE 3.3-13 (Continued)

TABLE NOTATIONS

\*At all times.

\*\*During WASTE GAS HOLDUP SYSTEM operation.

\*\*\*With irradiated fuel in the storage pool.

ACTION STATEMENTS

- ACTION 35 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment provided best efforts are made to repair the instrument and that prior to initiating the release:
- a. At least two independent samples of the tank's contents are analyzed, and
  - b. At least two technically qualified members of the facility staff independently verify the release rate calculations and discharge valve lineup;
- ACTION 36 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided best efforts are made to repair the instrument and that the flow rate is estimated at least once per 4 hours. For the waste gas holdup tank this action item is applicable only during periods of release. For the main condenser evacuation and turbine gland sealing systems, this action item applies only during release via the discharge silencer and only during turbine gland sealing operations and/or vacuum pump operation.
- ACTION 37 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided best efforts are made to repair the instrument and that grab samples are taken at least once per 12 hours and these samples are analyzed for gross activity within 24 hours. However, containment purging of radioactive effluents must be immediately suspended during this condition for the plant stack only.
- ACTION 38 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, operation of the WASTE GAS HOLDUP SYSTEM may continue provided best efforts are made to repair the instrument and that grab samples are collected at least once per 8 hours and analyzed within the following 4 hours for the onservice gas decay tank.

TABLE 3.3-13 (Continued)

ACTION STATEMENTS

- ACTION 39 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue provided best efforts are made to repair the instrument and that samples are continuously collected with auxiliary sampling equipment as required in Table 4.11-2.
- ACTION 40 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, operation of the WASTE GAS HOLDUP SYSTEM may continue provided best efforts are made to repair the instrument and that the system is sampled by either the remaining monitor or by a grab sample at least once per 4 hours and the oxygen concentration remains less than 2%. If there are no monitors OPERABLE, WASTE GAS HOLDUP SYSTEM operation may continue provided best efforts are made to return at least one channel to OPERABLE status and that a grab sample is taken and analyzed from the onservice gas decay tank at least once per 4 hours and the oxygen concentration remains less than 1%. With oxygen concentration exceeding 1%, reduce the oxygen concentration to less than 1% within 48 hours, or be in HOT-STANDBY within the next 6 hours.

### 3/4.11 RADIOACTIVE EFFLUENTS

#### 3/4.11.1 LIQUID EFFLUENTS

##### CONCENTRATION

##### LIMITING CONDITION FOR OPERATION

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3.11.1.1 The concentration of radioactive material released in liquid effluents to UNRESTRICTED AREAS (see Figure 5.1-3) shall be limited to the concentrations specified in 10 CFR Part 20, Appendix B, Table II, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to  $2 \times 10^{-4}$  microcurie/ml total activity.

APPLICABILITY: At all times.

##### ACTION:

With the concentration of radioactive material released in liquid effluents to UNRESTRICTED AREAS exceeding the above limits, immediately restore the concentration to within the above limits, and describe the events leading to this condition in the next Semiannual Radioactive Effluent Release Report.

##### SURVEILLANCE REQUIREMENTS

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4.11.1.1.1 Radioactive liquid wastes shall be sampled and analyzed according to the sampling and analysis program of Table 4.11-1.

4.11.1.1.2 The results of the radioactivity analyses shall be used in accordance with the methodology and parameters in the ODCM to assure that the concentrations at the point of release are maintained within the limits of Specification 3.11.1.1.



TABLE 4.11-1

RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

LIQUID RELEASE TYPE	SAMPLING FREQUENCY	MINIMUM ANALYSIS FREQUENCY	TYPE OF ACTIVITY ANALYSIS	LOWER LIMIT OF DETECTION (LLD) <sup>a</sup> (μCi/mL)
A. Batch Waste Release Tanks <sup>b,f,g,h,i</sup>	P Each Batch	P Each Batch	Principal Gamma Emitters <sup>c</sup>	5x10 <sup>-7</sup>
1. Boric Acid Condensate	P	M	I-131	1x10 <sup>-6</sup>
2. Waste Condensate	One Batch/M	M	Dissolved and Entrained Gases (Gamma Emitters)	1x10 <sup>-5</sup>
3. Laundry Waste	P Each Batch	M Composite <sup>d</sup>	H-3	1x10 <sup>-5</sup>
4. Turbine Building Industrial Waste Sumps*	P Each Batch	Q Composite <sup>d</sup>	Sr-89, Sr-90	5x10 <sup>-8</sup>
5. Dry Cooling Tower Sumps #1 and #2*			Fe-55	1x10 <sup>-6</sup>
6. Regenerative Waste				
7. Filter Flush				
8. Waste				

<sup>a</sup>When release from this source is batch in nature.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-38

LOUISIANA POWER AND LIGHT COMPANY  
WATERFORD STEAM ELECTRIC STATION, UNIT 3

DOCKET NO. 50-382

1.0 INTRODUCTION

By applications dated August 20 and October 1, 1986, Louisiana Power and Light Company (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-38) for the Waterford Steam Electric Station, Unit 3. The proposed changes would: (1) revise the reporting requirements for release of liquid radioactive effluents to unrestricted areas; (2) revise the action to be taken in the event that the liquid radioactive effluent monitoring instrumentation is inoperable and, (3) revise the action to be taken in the event that the gaseous radioactive effluent monitoring instrumentation is inoperable.

2.0 DISCUSSION

The proposed changes to the technical specifications requested by the licensee are in three areas, as described below.

2.1 Liquid Radioactive Effluent Release Reporting (NPF-38-33)

The proposed change would modify Technical Specification 3.11.1, "Liquid Effluents," which addresses the release of liquid radioactive effluents to unrestricted areas, by adding an additional action requirement to "describe the events leading to this condition in the next Semiannual Radioactive Effluent Release Report."

2.2 Liquid Radioactive Effluent Monitoring Instrumentation (NPF-38-49)

Technical Specification 3.3.3.10, "Radioactive Liquid Effluent Monitoring Instrumentation," provides for operable liquid radioactive effluent monitoring instrumentation to ensure that the release of liquid effluents does not exceed specified limits.

The proposed changes would revise Action Statement "b" to Technical Specification 3.3.3.10 by providing a 30-day time period to restore any required monitoring instrumentation (listed in Table 3.3.-12) to Operable status or, if unsuccessful, to explain in the next Semiannual Radioactive Effluent Release Report why the instrumentation was not restored in the specified time.

Also, Action Statements 28, 29 and 30 to Table 3.3-12 will be modified by removing any reference to a specified time period and allowing releases to continue as long as the Actions of Table 3.3-12 are continued and best efforts are made to repair the required instrumentation. In addition, the last part of Action Statement 28 will be deleted since it is already clear that releases through a specific pathway may not occur if the two conditions of this Action Statement cannot be satisfied.

### 2.3 Gaseous Radioactive Effluent Monitoring Instrumentation (NPF-38-50)

Technical Specification 3.3.3.11, "Radioactive Gaseous Effluent Monitoring Instrumentation," provides for operable gaseous radioactive effluent monitoring instrumentation to ensure that the release of gaseous effluents does not exceed specified limits.

The proposed changes would revise Action Statements "a" and "b" to Technical Specification 3.3.3.11 to add an option that would return the plant to compliance with the LCO and to specify a 30-day time period in which to restore an inoperable instrument to Operable status before it must be reported in the Semiannual Radioactive Effluent Release Report.

Also, Action Statements 35, 36, 37, 38, 39 and 40 to Table 3.3.12 have been modified by removing any reference to a specified time period and allowing releases to continue as long as the specified Actions of Table 3.3-13 are continued and best efforts are made to repair the required instrumentation.

### 3.0 EVALUATION

The proposed changes to the Technical Specifications requested by the licensee and described in three areas above, are evaluated below.

#### 3.1 Liquid Radioactive Effluent Release Reporting (NPF-38-33)

As the proposed change to Technical Specification 3.11.1 imposes another reporting requirement on the licensee and is strictly administrative in nature, the staff concludes that adding the additional Action requirement to "describe the events leading to this condition in the next Semiannual Radioactive Effluent Release Report" is acceptable.

#### 3.2 Liquid Radioactive Effluent Monitoring Instrumentation (NPF-38-49)

The proposed changes to Technical Specification 3.3.3.10 and Action Statements 28, 29 and 30 to Table 3.3-12 are consistent with the guidance of NUREG-0472, "Standard Radiological Effluent Technical Specifications for PWRs," in which the intent is to eliminate the need for a Licensee Event Report simply because the required instrumentation could not be restored to Operable status within the specified time. The concentration of radionuclides will remain within the limits specified in Technical Specification 3.11.1 and 10 CFR Part 20, Appendix B because the proposed Actions specified in Table 3.3-12 prior to and during any liquid effluent releases are continued. This will ensure that the levels of radioactive materials

in bodies of water in unrestricted areas will not result in exposures to any member of the general public in excess of 3 millirems to the total body or 10 millirems to any organ in accordance with 10 CFR Part 50, Appendix I, Section II.A.

As these changes meet the intent of 10 CFR Part 50, Appendix A, General Design Criteria 60, 63 and 64, which require a means to control and monitor all radioactive storage areas and releases to the environment during normal operation, including operational occurrences and postulated accidents, the staff finds them acceptable.

### 3.3 Gaseous Radioactive Effluent Monitoring Instrumentation (NPF-38-50)

The proposed changes to Technical Specification 3.3.3.11 and Action Statements 35, 36, 37, 38, 39 and 40 to Table 3.3-13 are consistent with the guidance of NUREG-0472, "Standard Radiological Effluent Technical Specifications for PWRs," in which the intent is to eliminate the need for a Licensee Event Report simply because the required instrumentation could not be restored to operable status within the specified time. By continuing to comply with the Action Statements of Table 3.3-13, the radioactive gaseous effluents released would not result in the exposure of a member of the public to an average concentration exceeding the limits specified in Appendix B of 10 CFR Part 20. In addition, this proposed change is consistent with the requirements of Appendix I to 10 CFR Part 50 by keeping the annual dose from gaseous effluents to individuals in unrestricted areas less than 10 millirads for gamma radiation and 20 millirads for beta radiation.

As these changes meet the intent of 10 CFR Part 50, Appendix A, General Design Criteria 60, 63 and 64, which require a means to control and monitor all radiological storage areas and releases to the environment during normal operation, including anticipated operational occurrences and postulated accidents, the staff finds them acceptable.

### 4.0 CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Administrator, Nuclear Energy Division, Office of Environmental Affairs, State of Louisiana of the proposed determination of no significant hazards consideration. No comments were received.

### 5.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation or use of facility components located within the restricted area. The staff has determined that the amendment involves no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendment involves no significant hazards consideration, and there has been no public comment on such findings. Accordingly,

the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 6.0 CONCLUSION

Based upon our evaluation of the proposed changes to the Waterford 3 Technical Specifications, we have concluded that: there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the proposed changes are acceptable, and are hereby incorporated into the Waterford 3 Technical Specifications.

Dated: March 15, 1987

Principal Contributor: C. Nichols