NOV 27 1984

Docket Nos.: 50-382

Mr. R. S. Leddick
Senior Vice President - Nuclear Operations
Louisiana Power and Light Company
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STRIBUTION
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Dear Mr. Leddick:

Subject: Waterford Steam Electric Station, Unit 3 Draft License

As you are aware, the staff is considering issuance of a license for the Waterford Steam Electric Station, Unit 3. Enclosed is a draft copy of the license. It is provided for your information and comment to insure that it accurately reflects the commitments required of you as described in the FSAR, SER, and other documentation.

You should note that this draft does not as yet contain any requirements which may result from the staff's review of either the adequacy of the common foundation-basemat or the 23 issues indicated in the June 13, 1984, letter from D. G. Eisenhut to J. M. Cain. Issuance of the license is subject to satisfactory resolution of these matters.

We request that you examine the draft license and provide any comments in writing by November 29, 1984.

For any questions regarding this draft license, contact the Waterford Project Manager, J. Wilson at (301) 492-7702.

Sincerely,

/s/

Thomas M. Novak, Assistant Director
for Licensing
Division of Licensing

Enclosure:
As stated

cc: See next page

Previous concurrences concurred on by*:

DL:LB#3	DL:LB#3	DL:LB#3	DL:AD/L	DL:DIR
*JWilson/yt	*JLee	*GWKnighton	*TMNovak	*DGEisenhut
10/04/84	10/04/84	10/04/84	11/15/84	11/21/84

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DRAFT

LOUISIANA POWER AND LIGHT COMPANY

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

FACILITY OPERATING LICENSE

License No. NPF-26

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for license filed by the Louisiana Power and Light Company (licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;

   B. Construction of the Waterford Steam Electric Station, Unit 3 (facility), has been substantially completed in conformity with Construction Permit No. CPPR-103 and the application as amended, the provisions of the Act, and regulations of the Commission;

   C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission (except as exempted from compliance in Section 1.I below);

   D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I (except as exempted from compliance in Section 1.I. below);

   E. The Louisiana Power & Light Company is technically qualified to engage in the activities authorized by this operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;

   F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;

G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;

H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs, and after considering available alternatives, the issuance of the Facility Operating License No. NPF-26, subject to the conditions for protection of the environment set forth in the Environmental Protection Plan attached as Appendix B, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and

I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70, except that an exemption to the provisions of 10 CFR 70.24 is granted as described in Supplement No. 8 to the Safety Evaluation Report. This exemption is authorized under 10 CFR 70.24(d) and will not endanger life or property or the common defense and security and is otherwise in the public interest.

2. Based on the foregoing, Facility Operating License No. NPF-26 is hereby issued to the Louisiana Power and Light Company (licensee) to read as follows:

A. This license applies to the Waterford Steam Electric Station, Unit 3, a pressurized water reactor and associated equipment (the facility), owned by Louisiana Power and Light Company (the licensee). The facility is located on the licensee's site in St. Charles Parish, Louisiana and is described in the Louisiana Power and Light Company Final Safety Analysis Report as amended, and the Environmental Report as amended.

B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Louisiana Power and Light Company (LP&L):

1. Pursuant to Section 103 of the Act and 10 CFR Part 50, to possess, use and operate the facility at the designated location in St. Charles Parish, Louisiana in accordance with the procedures and limitations set forth in this license;

2. Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended through Amendment 36;

3. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed sources for reactor startup, as sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

4. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

5. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility authorized herein.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

1. Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3390 megawatts thermal (100% power) in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational tests, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license. Pending Commission approval, this license is restricted to power levels not to exceed 5 percent of full power (169 megawatts thermal).

2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in the attached Appendix A and the Environmental Protection Plan contained in the attached Appendix B, are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. Antitrust Conditions

The licensee shall comply with the antitrust conditions in Appendix C to this license.

4. Broad Range Toxic Gas Detectors (Section 2.2.1, SSER 6*)

By May 1, 1985, the licensee shall have an operable detection system, or equivalent protective measures, capable of detecting and indicating the presence of toxic gases at the control room air intakes. Prior to startup following the first refueling outage, the licensee shall propose technical specifications for the system for inclusion in Appendix A to this license.

5. Initial Inservice Inspection Program (Section 6.6, SSER 5)

By June 1, 1985, the licensee must submit an initial inservice inspection program for staff review and approval.

6. Environmental Qualification (Section 3.11, SSER 8)

(a) Prior to March 31, 1985, the licensee shall environmentally qualify all electrical equipment according to the provisions of 10 CFR 50.49.

(b) Prior to December 31, 1984, the licensee shall provide an aging analysis for all non-metallic components in safety-related mechanical equipment located in a harsh environment.

7. Axial Fuel Growth (Section 4.2, SSER 5)

Prior to startup following the first refueling outage, the licensee shall provide surveillance results or modifications which assure that the shoulder gap clearance is adequate.

8. Emergency Preparedness (Section 13.3, SSER 8)

In the event that the NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of emergency preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

9. Fire Protection (Section 9.5.1, SSER 8)

a. The licensee shall maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility through Amendment 36 and as approved in the SER through Supplement 8, subject to provisions b & c below.

b. The licensee may make no change to features of the approved fire protection program which would decrease the level of fire protection in the plant without prior approval of the Commission. To make such a change the licensee must submit an application for license amendment pursuant to 10 CFR 50.90.

c. The licensee may make changes to features of the approved fire protection program which do not decrease the level of fire protection without prior Commission approval, provided:

(1) such changes do not otherwise involve a change in a license condition or technical specification or result in an unreviewed safety question (see 10 CFR 50.59).

(2) such changes do not result in failure to complete the fire protection program approved by the Commission prior to license issuance.

The licensee shall maintain, in an auditable form, a current record of all such changes including an analysis of the effects of the change on the fire protection program and shall make such records available to NRC inspectors upon request. All changes to the approved program made without prior Commission approval shall be reported annually to the Director of the Office of Nuclear Reactor Regulation, together with supporting analyses.

d. The licensee shall install battery-operated smoke detectors in the Control Room main control panels prior to initial criticality and shall replace those detectors with detectors that are installed in accordance with NFPA 72E prior to startup following the first refueling outage.

e. The licensee shall perform a spurious signal analysis for the case of a fire in any plant area (including the control room and cable vault) combined with a loss of offsite power by December 1, 1984, and shall complete any resulting modifications prior to startup following the first refueling outage, but in any case not later than June 1, 1987.

f. The licensee shall provide neutron flux indication at LCP-43 which is electrically independent of the control room and cable vault prior to start-up following the first refueling outage, but in any case not later than June 1, 1987.

g. The licensee shall provide a continuous fire watch in the relay room at the isolation panel from initial criticality until acceptable resolution of adverse effects, if any, of the loss of this panel on safe shutdown.

10. Initial Test Program (Section 14, SER)

The licensee shall conduct the post-fuel-loading initial test program described in Chapter 14 of the FSAR, as amended, without making any major modifications unless such modifications have prior NRC approval. Major modifications are defined as:

a. elimination of any safety-related test*

b. modification of objectives, test method, or acceptance criteria for any safety-related test

c. performance of any safety-related test at a power level different from that stated in the FSAR by more than 5 percent of rated power

d. failure to satisfactorily complete the entire initial startup test program by the time core burnup equals 120 effective full power days

e. deviation from initial test program administrative procedures or quality assurance controls described in the FSAR

f. delays in test program in excess of 30 days (14 days if power level exceeds 50 percent), concurrent with power operation. If continued power operation is desired during a delay, the licensee shall provide justification that adequate testing has been performed and evaluated to demonstrate that the facility can be operated at the planned power level with reasonable assurance that the health and safety of the public will not be endangered.

11. Emergency Response Capabilities (Section 22, SSER 8)

Prior to May 1, 1985, the licensee shall submit for staff review and approval the Detailed Control Room Design Review Summary Report, including a description of the function and task analysis. Approval of this submittal will also serve to satisfy the task analysis requirements associated with the Procedures Generation Package.

*Safety-related tests are those tests which verify the design, construction, and operation of safety-related systems, structures, and equipment.

12. Reactor Coolant System (RCS) Depressurization Capability (Section 5.4.3, SSER 8)

By March 1, 1985, the licensee shall submit the results of confirmatory tests regarding the depressurization capability of the auxiliary pressurizer spray (APS) system. This information must demonstrate that the APS system can perform the necessary depressurization to meet the steam generator single-tube rupture accident acceptance criteria (SRP 15.6.3). Should the test results fail to demonstrate that the acceptance criteria are met, the licensee must provide for staff review and approval, justification for interim operation, and a schedule for corrective actions.

13. Response to Salem ATWS Event (Section 7.2.9, SSER 8)

The licensee shall submit responses and implement the requirements of Generic Letter 83-28 on a schedule which is consistent with that given in the licensee's letter of May 30, 1984.

14. Spent Fuel Storage Racks (Section 9.2 SSER 8)

The applicant shall confirm the presence of the Boraflex at all specified design locations in the spent fuel pool rack array prior to startup following the first refueling outage. The spent fuel storage racks may be used prior to satisfactory completion of the confirmatory tests, provided fuel assemblies are stored only in alternate rows and columns in the racks with center-to-center spacing between fuel assemblies of at least 20.5 inches.

15. Qualification of Personnel (Section 13.1.3, SSER 8)

The licensee shall have on each shift operators who meet the requirements described in Attachment 2. Attachment 2 is hereby incorporated into this license.

16. Post Accident Monitoring Instrumentation (7.5.2, SSER 8)

Prior to startup following the first refueling ontage, the licensee shall complete any required modifications to provide compliance with R.G. 1.97, unless the deviations are approved by the staff.

D. (Reserved for Part 50 Exemptions, as required)

E. The licensee shall fully implement and maintain in effect all the provisions of the Commission-approved physical security, guard training and qualification and safeguards contingency plans, including amendments made pursuant to the authority of 10 CFR Section 50.54(p). The approved plans, which contain Safeguards Information as described in 10 CFR Section 73.21, are entitled "Site Security Plan Waterford Steam Electric Station Unit No. 3", Revision 6, dated July 6, 1981, Revision 7 dated February 23, 1983, Revision 8 dated April 10, 1984 (transmittal letter dated April 11, 1984); "Waterford 3 Steam Electric Station Safeguards Contingency Plan," dated February 1, 1980 as revised July 1, 1980, Revision 2 dated March 14, 1983 and Revision 3, dated January 16, 1984; transmittal letter dated January 12, 1984; "Waterford Generating Station Guard Training & Qualification Plan" dated February 1, 1980, as revised by pages submitted by letter dated April 23, 1981, Revision 2 dated December 19, 1983, transmittal letter dated December 16, 1983.

F. The licensee shall report any violations of the requirements contained in Section 2, Items C.(1), C.(3) through C.(15) of this license within twenty-four (24) hours. Initial notification shall be made in accordance with the provisions of 10 CFR 50.72 with written follow-up in accordance with the procedures described in 10 CFR 50.73 (b), (c) and (e).

G. The licensee shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

H. This license is effective as the date of issuance and shall expire at midnight on

FOR THE NUCLEAR REGULATORY COMMISSION

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:
1. Attachment 1
2. Attachment 2
3. Appendix A (Technical Specifications) (NUREG-0983)
4. Appendix B (Environmental Protection Plan)
5. Appendix C (Antitrust Conditions)

Date of Issuance:

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ATTACHMENT 2

Operating Staff Experience Requirements

LP&L shall have a licensed senior operator on each shift who has had at least six months of hot operating experience on a pressurized water reactor, including at least six weeks at power levels greater then 20% of full power, and who has had startup and shutdown experience. For those shifts where such an indivdual is not available on the plant staff, an advisor shall be provided who has had at least four years of power plant experience, including two years of nuclear plant experience, and who has had at least one year of experience on shift as a licensed senior operator at a similar type facility. Use of advisors who were licensed only at the RO level will be evaluated on a case-by-case basis. Advisors shall be trained on plant procedures, technical specifications and plant systems, and shall be examined on these topics at a level sufficient to to assure familiarity with the plant. For each shift, the remainder of the shift crew shall be trained in the role of the advisors. The training of the advisors and remainder of the shift crew shall be completed prior to exceeding 5% power. Prior to exceeding 5% power, LP&L shall certify to the NRC the names of the advisors who have been examined and have been determined to be competent to provide advice to the operating shifts. These advisors shall be retained until the experience levels identified in the first sentence above have been achieved. The NRC shall be notified at least 30 days prior to the date LP&L proposes to release the advisors from further service.