Docket

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

50-528/529/530

December 23, 1998

Mr. James M. Levine
Senior Vice President, Nuclear
Arizona Public Service Company
Post Office Box 53999
Phoenix, Arizona 85072-3999

SUBJECT: ISSUANCE OF AMENDMENTS FOR THE PALO VERDE NUCLEAR GENERATING STATION UNIT NO. 1 (TAC NO. MA3793) AND UNIT NO. 2 (TAC NO. MA3794)

Dear Mr. Levine:

The Commission has issued the enclosed Amendment No. 119 to Facility Operating License No. NPF-41 and Amendment No. 119 to Facility Operating License No. NPF-51 for the Palo Verde Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated October 6, 1998.

This amendments revise TS 3.3.1, "Reactor Protective System (RPS) Instrumentation – Operating," and TS 3.3.2, "Reactor Protective System (RPS) Instrumentation – Shutdown." The amendments clarify the power level threshold at which certain RPS instrumentation trips must be enabled and may be bypassed, and clarify that this level is a percentage of the neutron flux at rated thermal power (RTP). The bypass power level, 1E-4% RTP, is specified as logarithmic power instead of thermal power.

The NRC approved these changes for Palo Verde Unit 3 on an exigent basis as indicated in its letter dated October 19, 1998. The exigent TS amendment resulted in TS pages with notes specifying different requirements between Unit 3 and Units 1 and 2. These amendments remove these notes regarding Unit 3 from the affected TS pages so that all Units have the same TS.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated October 6, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:

1.1 Definitions

LEAKAGE (continued)	c. Pressure Boundary LEAKAGE LEAKAGE (except SG LEAKAGE) through a nonisolable fault in an RCS component body, pipe wall, or vessel wall.
MODE	A MODE shall correspond to any one inclusive combination of core reactivity condition, power level, cold leg reactor coolant temperature, and reactor vessel head closure bolt tensioning specified in Table 1.1-1 with fuel in the reactor vessel.
NEUTRON RATED THERMAL POWER (NRTP)	The indicated neutron flux at RTP.
OPERABLE – OPERABILITY	A system, subsystem, train, component, or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified safety function(s) and when all necessary attendant instrumentation, controls, normal or emergency electrical power, cooling and seal water, lubrication, and other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its specified safety function(s) are also capable of performing their related support function(s).
PHYSICS TESTS	PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are: a. Described in Chapter 14, Initial Test Program of the UFSAR; b. Authorized under the provisions of 10 CFR 50.59; or c. Otherwise approved by the Nuclear Regulatory Commission.

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.3.1.6	----------NOTE---------- Not required to be performed until 12 hours after THERMAL POWER ≥ 15% RTP. ---------- Verify linear power subchannel gains of the excore detectors are consistent with the values used to establish the shape annealing matrix elements in the CPCs.	31 days
SR 3.3.1.7	----------NOTES---------- 1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC. 2. Not required to be performed for logarithmic power level channels until 2 hours after reducing logarithmic power below 1E-4% NRTP. ---------- Perform CHANNEL FUNCTIONAL TEST on each channel.	92 days
SR 3.3.1.8	----------NOTE---------- Neutron detectors are excluded from the CHANNEL CALIBRATION. ---------- Perform CHANNEL CALIBRATION of the power range neutron flux channels.	92 days

(continued)

Table 3.3.1-1 (page 1 of 3)
Reactor Protective System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Variable Over Power	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.6 SR 3.3.1.7 SR 3.3.1.8 SR 3.3.1.9 SR 3.3.1.13	Ceiling ≤ 111.0% RTP Band ≤ 9.9% RTP Incr. Rate ≤ 11.0%/min RTP Decr. Rate > 5%/sec RTP
2. Logarithmic Power Level – High[(a)]	2	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.12 SR 3.3.1.13	≤ 0.011% NRTP
3. Pressurizer Pressure – High	1,2	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.13	≤ 2388 psia
4. Pressurizer Pressure – Low	1,2	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.12 SR 3.3.1.13	≥ 1821 psia
5. Containment Pressure – High	1,2	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.13	≤ 3.2 psig
6. Steam Generator #1 Pressure – Low	1,2	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.13	≥ 890 psia
7. Steam Generator #2 Pressure – Low	1,2	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.13	≥ 890 psia

(continued)

(a) Trip may be bypassed when logarithmic power is > 1E-4% NRTP. Bypass shall be automatically removed when logarithmic power is ≤ 1E-4% NRTP.

Table 3.3.1-1 (page 3 of 3)
Reactor Protective System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
14. Local Power Density – High[(b)]	1,2	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.10 SR 3.3.1.11 SR 3.3.1.12 SR 3.3.1.13	≤ 21.0 kW/ft
15. Departure From Nucleate Boiling Ratio (DNBR) – Low[(b)]	1,2	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.10 SR 3.3.1.11 SR 3.3.1.12 SR 3.3.1.13	≥ 1.30

(b) Trip may be bypassed when logarithmic power is < 1E-4% NRTP. Bypass shall be automatically removed when logarithmic power is ≥ 1E-4% NRTP.

Table 3.3.2-1
Reactor Protective System Instrumentation - Shutdown

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALVE
1. Logarithmic Power Level-High[d]	3[a], 4[a], 5[a]	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.5	≤ 0.011% NRTP[c]
2. Steam Generator #1 Pressure-Low[b]	3[a]	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.4 SR 3.3.2.5	≥ 890 psia
3. Steam Generator #2 Pressure-Low[b]	3[a]	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.4 SR 3.3.2.5	≥ 890 psia

(a) With any Reactor Trip Circuit Breakers (RTCBs) closed and any control element assembly capable of being withdrawn.

(b) The setpoint may be decreased as steam pressure is reduced, provided the margin between steam pressure and the setpoint is maintained ≤ 200 psig. The setpoint shall be automatically increased to the normal setpoint as steam pressure is increased.

(c) The setpoint must be reduced to ≤ 1E-4% NRTP when less than 4 RCPs are running.

(d) Trip may be bypassed when logarithmic power is > 1E-4% NRTP. Bypass shall be automatically removed when logarithmic power is ≤ 1E-4% NRTP.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. NPF-41

AND AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. NPF-51

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

PALO VERDE NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. STN 50-528 AND STN 50-529

1.0 INTRODUCTION

By letter dated October 6, 1998, Arizona Public Service Company (APS or the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License Nos. NPF-41, NPF-51, and NPF-74) for Palo Verde Nuclear Generating Station (PVNGS), Units 1, 2 and 3. The Arizona Public Service Company submitted this request on behalf of itself, the Salt River Project Agricultural Improvement and Power District, Southern California Edison Company, El Paso Electric Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority.

The proposed changes would modify TS 3.3.1, "Reactor Protective System (RPS) Instrumentation – Operating," and TS 3.3.2, "Reactor Protective System (RPS) Instrumentation – Shutdown," to clarify the power level threshold at which certain RPS instrumentation trips must be enabled and may be bypassed, and clarify that this level is a percentage of the neutron flux at rated thermal power (RTP). The bypass power level, 1E-4% RTP, would be specified as logarithmic power instead of thermal power.

The NRC approved these changes for Palo Verde Unit 3 on an exigent basis as indicated in its letter dated October 19, 1998. The exigent TS amendment resulted in TS pages with notes specifying different requirements between Unit 3 and Units 1 and 2. This amendment removes these notes from the affected TS pages.

2.0 DISCUSSION

Footnotes (a) and (b) in TS Table 3.3.1, "Reactor Protective System (RPS) Instrumentation – Operating," and footnote (d) in TS 3.3.2 Table, "Reactor Protective System (RPS) Instrumentation – Shutdown," identify operating bypass permissive and enable bistable values. The proposed amendments to the PVNGS TS would replace the words "THERMAL POWER" with "logarithmic power" for the 1E-4% RTP level threshold in these footnotes, and also in

Surveillance Requirement (SR) 3.3.1.7 Note 2. The proposed amendments would also replace "RTP" with "NRTP," in Table 3.3.1-1 footnotes (a) and (b), SR 3.3.1.7 Note 2, and Table 3.3.2-1 footnotes (c) and (d). In addition, the proposed amendments would add a definition for NRTP (neutron rated thermal power) in TS Section 1.1 as the indicated neutron flux at RTP, and specify NRTP as the "ALLOWABLE VALUE" parameter for the logarithmic power level – high trip in Table 3.3.1-1.

TS 3.3.1, "Reactor Protective System (RPS) Instrumentation – Operating," provides for the operability, during plant operation, of instruments necessary to initiate a reactor trip that would protect the plant against anticipated operational occurrences and assist the engineered safety features (ESF) systems in mitigating accidents. Those instruments are specified in Table 3.3.1-1.

The logarithmic power level - high trip specified in TS Table 3.3.1-1 is designed to protect the integrity of the fuel cladding and help protect the reactor coolant pressure boundary in the event of an unplanned criticality from a shutdown condition. The purpose of footnote (a) is to allow the logarithmic power trip to be bypassed when neutron power is above 1E-4% NRTP, and require the trip to be automatically enabled when neutron power is at or below 1E-4% NRTP. This is permitted because the logarithmic power trip is not needed unless neutron power (indicated by logarithmic power) is 1E-4% NRTP or below. As described in the Bases for TS 3.3.1, other trips provide adequate protection for events originating when power is above 1E-4% NRTP.

The local power density (LPD) - high and departure from nucleate boiling ratio (DNBR) - low trips in TS Table 3.3.1-1 are designed to provide plant protection during certain anticipated operational occurrences and assist the ESF systems in the mitigation of certain accidents, as described in the Bases for TS 3.3.1. The purpose of footnote (b) is to allow the LPD-high and DNBR-low trips to be bypassed when neutron power is below 1E-4% NRTP, and require the trips to be automatically enabled when neutron power (indicated by logarithmic power) is at or above 1E-4% NRTP. This is permitted because, as described in the Bases for TS 3.3.1, plant conditions when power is below 1E-4% NRTP do not warrant the trip protection of these trips.

SR 3.3.1.7 requires a channel functional test be performed on each RPS channel at a frequency of 92 days to assure that the instruments will be operable. Note 2 of this SR allows the functional test of the logarithmic power level channels to be deferred until two hours after reducing power below 1E-4% NRTP, since the logarithmic power trip is not needed unless neutron power (indicated by logarithmic power) is at 1E-4% NRTP or below.

TS 3.3.2, "Reactor Protective System (RPS) Instrumentation – Shutdown," provides for the operability, during plant shutdown, of instruments necessary to initiate a reactor trip that would protect the plant against anticipated operational occurrences and assist the ESF systems in mitigating accidents. Those instruments are specified in Table 3.3.2-1.

The logarithmic power level - high trip specified in TS Table 3.3.2-1 is designed to protect the integrity of the fuel cladding and help protect the reactor coolant pressure boundary in the event of an unplanned criticality from a shutdown condition. The purpose of the footnote (d) is to

allow the logarithmic power level - high trip to be bypassed when neutron power is above 1E-4% NRTP, and require the trip to be automatically enabled when neutron power is at or below 1E-4% NRTP. This is permitted because the logarithmic power level - high trip is not needed unless neutron power (indicated by logarithmic power) is 1E-4% NRTP or below.

3.0 EVALUATION

As stated in the previous section, the proposed TS amendments would replace the words "THERMAL POWER" with "logarithmic power" and replace RTP with NRTP in TS 3.3.1 and TS 3.3, and add a definition of NRTP to TS 1.1. As described above, the purpose of the 1E-4% NRTP threshold is to (1) specify the power, below which, the logarithmic power level trip is required to be operable and surveilled, and (2) specify the power, above which, the LPD and DNBR trips are required to be operable. For all of these purposes, the appropriate power threshold should be logarithmic power, which is the power indicated on the logarithmic nuclear instrumentation, and not thermal power. Thermal power is defined in TS Section 1.1 as the total reactor heat transfer rate to the reactor coolant, and would include decay heat. Thermal power would therefore not drop to 1E-4% RTP for a number of years after shutdown, and would not provide the plant protective function correlation required at 1E-4% NRTP. Since "THERMAL POWER" will not decrease to less than or equal to 1E-4% RTP for normal duration plant outages, TS Table 3.3.1-1, note "b," would require the LPD/DNBR trip bypasses to be removed during planned startup when the plant enters Mode 2. This condition is expected to produce a trip signal as soon as the trip bypasses are removed. Therefore strict adherence to the notes as currently written would preclude plant startups.

The PVNGS Updated Final Safety Analysis Report (UFSAR) Section 7.2.1.1.2.3 states that the excore neutron flux instrumentation provides the input signal to the RPS for the logarithmic power level - high trip and to the core protection calculator (CPC) for use in calculations for LPD-high and DNBR-low trips. Further, UFSAR Section 15.4.1.3 states that a trip generated at 1E-4% power level (when the CPC bypass is automatically removed) would cause a decrease in fission (neutron) power before the point of adding sensible heat is reached. Also, UFSAR Section 15.4.1.4 states that a reactor trip on high logarithmic power is generated before core power reaches the point of adding sensible heat. In all of these sections, it is implicit that the power being described is neutron flux power, as indicated by logarithmic power, and not thermal power, which is defined as heat transfer from the reactor core to the coolant. In addition, the TS Bases for LCO 3.3.1 for the LPD–high trip and the DNBR–low trip state that the 1E-4% RTP threshold level is "sensed by the logarithmic nuclear instrumentation."

Therefore, logarithmic power, which measures neutron flux, does provide the plant protective function correlation required at 1E-4% NRTP for the required trips as required by safety analyses. The logarithmic power level of 1E-4% NRTP nominally correlates to the neutron flux measured by the excore neutron instrumentation that is 1E-4% of the neutron flux at 100% RTP (3876 MWt) measured by the excore neutron instrumentation. Since neutron flux is, by design, the correct input process variable for the operating bypass permissive and enable bistable values described in footnotes (a) and (b) of TS Table 3.3.1-1, footnote (d) of TS Table 3.3.2-1, and Note 2 to SR 3.3.1.7, the change to replace "THERMAL POWER" with logarithmic power is acceptable to the staff.

The proposed amendments would also replace "RTP" with "NRTP," in Table 3.3.1-1 footnotes (a) and (b), SR 3.3.1.7 Note 2, and Table 3.3.2-1 footnotes (c) and (d). A definition would be added for NRTP (neutron rated thermal power) in TS Section 1.1 as the indicated neutron flux at RTP. These clarifications will reflect the fact that the logarithmic power level of 1E-4% is not a percentage of the "total reactor core heat transfer rate to the reactor coolant of 3876 MWt," as RTP is defined in TS 1.1, but is instead a percentage of the indicated neutron flux at RTP. This proposed change is acceptable to the staff.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arizona State official was notified of the proposed issuance of the amendments for the PVNGS units. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (63 FR 59586). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Fields, PDIV-2/NRR

Date: December 23, 1998

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. NPF-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated October 6, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 119, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 23, 1998

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-528
and STN 50-529

Enclosures: 1. Amendment No. 119 to NPF-41
2. Amendment No. 119 to NPF-51
3. Safety Evaluation

cc w/encls: See next page

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ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. NPF-41

AND AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NOS. STN 50-528 AND STN 50-529

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

REMOVE	INSERT
1.1-5	1.1-5
3.3.1-6	3.3.1-6
3.3.1-8	3.3.1-8
3.3.1-10	3.3.1-10
3.3.2-5	3.3.2-5

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 119, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 23, 1998

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