



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 21, 1993

Docket Nos. 50-528, 50-529
and 50-530

Mr. William F. Conway
Executive Vice President, Nuclear
Arizona Public Service Company
Post Office Box 53999
Phoenix, Arizona 85072-3999

Dear Mr. Conway:

SUBJECT: ISSUANCE OF AMENDMENTS FOR THE PALO VERDE NUCLEAR GENERATING
STATION UNIT NO. 1 (TAC NO. M86439), UNIT NO. 2 (TAC NO. M86440),
AND UNIT NO. 3 (TAC NO. M86441)

The Commission has issued the enclosed Amendment No. 71 to Facility Operating License No. NPF-41, Amendment No. 57 to Facility Operating License No. NPF-51, and Amendment No. 44 to Facility Operating License No. NPF-74 for the Palo Verde Nuclear Generating Station, Unit Nos. 1, 2, and 3, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated May 20, 1993.

The amendment adds a footnote to Technical Specification Surveillance Requirement 4.7.9 for each of the 3 units. The footnote explains the methodology used for selecting the test sample for snubber functional testing during the current outage for Unit 2 and the third outage for both Units 1 and 3 which was not in accordance with the requirements of Surveillance Requirement 4.7.9 but adequately demonstrated the operability of the snubber population. The change was requested on an emergency basis when you determined the APS interpretation and implementation of TS 4.7.9 did not meet the surveillance requirement.

With the issuance of these amendments, our Notice of Enforcement Discretion issued on May 14, 1993, is hereby cancelled.

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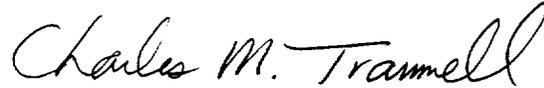
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Mr. William F. Conway

- 2 -

A copy of the related Safety Evaluation is also enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,



Charles M. Trammell, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 71 to NPF-41
2. Amendment No. 57 to NPF-51
3. Amendment No. 44 to NPF-74
4. Safety Evaluation

cc w/enclosures:
See next page

Mr. William F. Conway

- 2 -

June 21, 1993

A copy of the related Safety Evaluation is also enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Charles M. Trammell, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 71 to NPF-41
- 2. Amendment No. 57 to NPF-51
- 3. Amendment No. 44 to NPF-74
- 4. Safety Evaluation

cc w/enclosures:
See next page

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subject to change

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Arizona Public Service Company

Palo Verde

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 71
License No. NPF-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated May 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 71, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 21, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 71 TO FACILITY OPERATING LICENSE NO. NPF-41

DOCKET NO. STN 50-528

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 7-21

Insert

3/4 7-21

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All hydraulic and mechanical snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable on any system, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.*

a. Snubber Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-2. The visual inspection interval for each type of snubber shall be determined based upon the criteria provided in Table 4.7-2 and the first inspection interval determined using this criteria shall be based upon the previous inspection interval as established by the requirements in effect before Amendment No. 57.

*For functional testing performed during the third refueling outage, the selection of the snubber test samples was not performed in accordance with the requirements of the Specification. The methodology described in licensee correspondence dated May 20, 1993 was used for the sample selections for this testing and adequately demonstrated the operability of the snubber population.



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ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated May 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Part I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 57, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 21, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NO. STN 50-529

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 7-21

Insert

3/4 7-21

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All hydraulic and mechanical snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable on any system, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.*

a. Snubber Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-2. The visual inspection interval for each type of snubber shall be determined based upon the criteria provided in Table 4.7-2 and the first inspection interval determined using this criteria shall be based upon the previous inspection as interval established by the requirements in effect before Amendment No. 44.

*For functional testing performed during the fourth refueling outage, the selection of the snubber test samples was not performed in accordance with the requirements of the Specification. The methodology described in licensee correspondence dated May 20, 1993 was used for the sample selections for this testing and adequately demonstrated the operability of the snubber population.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44
License No. NPF-74

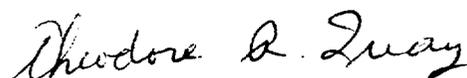
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated May 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 44, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 21, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. NPF-74

DOCKET NO. STN 50-530

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 7-21

Insert

3/4 7-21

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All hydraulic and mechanical snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable on any system, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.*

a. Snubber Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-2. The visual inspection interval for each type of snubber shall be determined based upon the criteria provided in Table 4.7-2 and the first inspection interval determined using this criteria shall be based upon the previous inspection as established by the requirements in effect before Amendment No. 30.

*For functional testing performed during the third refueling outage, the selection of the snubber test samples was not performed in accordance with the requirements of the Specification. The methodology described in licensee correspondence dated May 20, 1993 was used for the sample selections for this testing and adequately demonstrated the operability of the snubber population.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 71 TO FACILITY OPERATING LICENSE NO. NPF-41,
AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-51,
AND AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. NPF-74
ARIZONA PUBLIC SERVICE COMPANY, ET AL.
PALO VERDE NUCLEAR GENERATING STATION, UNIT NOS. 1, 2, AND 3
DOCKET NOS. STN 50-528, STN 50-529, AND STN 50-530

1.0 INTRODUCTION

By letter dated May 20, 1993, the Arizona Public Service Company (APS or the licensee) submitted a request for changes to the Technical Specifications (TS) for the Palo Verde Nuclear Generating Station, Units 1, 2, and 3 (Appendix A to Facility Operating License Nos. NPF-41, NPF-51, and NPF-74, respectively). The Arizona Public Service Company submitted this request on behalf of itself, the Salt River Project Agricultural Improvement and Power District, Southern California Edison Company, El Paso Electric Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority. The proposed changes would add a footnote to Surveillance Requirement 4.7.9 for each of the 3 units to state that the methodology used for selecting the test sample for snubber functional testing during the current outage for Unit 2 and the third outage for both Units 1 and 3 was not in accordance with the requirements of Surveillance Requirement 4.7.9 but adequately demonstrated the operability of the snubber population.

2.0 EVALUATION

Snubbers are devices which allow movement of piping systems to accommodate thermal effects but which restrain movement during dynamic events. The purpose of Technical Specification 3.7.9, "Snubbers," is to ensure that the structural integrity of the reactor coolant system and all other safety-related systems is maintained during and following a seismic or other event initiating dynamic loads.

In order to demonstrate the operability of the snubbers, Surveillance Requirement 4.7.9 requires that visual examination and functional tests be performed on the snubber. Functional test requirements are established based on a statistical approach for demonstrating the operability of the entire snubber population by performing actual tests on a sample population. Achieving acceptable results on the sample population demonstrate with sufficient confidence that the entire population is operable.

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To provide assurance of snubber functional reliability, the licensee's TS require one of the following three functional testing methods to be used: (1) Functionally test 10% of a snubber type, with an additional 10% tested for each functional failure, or (2) functionally test a representative sample size and determine the sample acceptance or rejection in accordance with the statistically-generated TS Figure 4.7-1, "Sampling Plan for Snubber Functional Test," or (3) functionally test a representative sample size of 55 snubbers and determine sample acceptance or rejection using the equation $N=55(1+C/2)$, where "C" is the number of snubbers found which do not meet the functional test acceptance criteria. In addition, the representative sample selected for the functional test sample plans shall be randomly selected from the snubbers of each type and reviewed before beginning the testing.

In correspondence with Region V, the licensee indicated that it had selected functional testing Sample Plan 2 for use at the Palo Verde Nuclear Generating Station (PVNGS). However, for each outage, the licensee tested a representative sample of mechanical snubbers (37) following Sample Plan 2 but tested one steam generator hydraulic snubber and one reactor coolant pump hydraulic snubber effectively following Sample Plan 1 for hydraulic snubbers.

While reviewing the snubber testing on Unit 2, it was discovered that the licensee had divided the snubbers into 5 groups. These are: (1) Pacific Scientific Arrester (PSA) 1/4 and 1/2 size Mechanical Snubbers (small), (2) PSA 1, 3, and 10 size Mechanical Snubbers (medium), (3) PSA 35 and 100 size Mechanical Snubbers (large), (4) Steam Generator Hydraulic Snubbers, and (5) Reactor Coolant Pump Hydraulic Snubbers. The surveillance method that the licensee indicated it had selected states that a sample of 37 snubbers of each type is to be tested. The TS definition of type requires that each of these five groups be considered as a snubber type. Since each group was not treated as a type, the licensee tested far fewer snubbers than the words in the TS would require. Upon discovery, an evaluation of the snubber testing program was performed and concluded that the testing satisfactory demonstrated operability of the snubbers, however, it was determined that the program was not being conducted in full compliance with the Technical Specifications. As such, on May 12, 1993, PVNGS Unit 1, 2, and 3 entered the ACTION statement for Limiting Condition for Operation 3.7.9. This ACTION statement allows 72 hours to return snubbers to OPERABLE status. At the end of this period, TS 3.0.3 would have had to be entered and plant shutdowns for both Units 1 and 3 would have had to commence. Unit 2, which is currently in an outage, would have been prohibited from changing modes.

By letter dated May 14, 1993, as supplemented by a further letter of the same date, the licensee requested a Notice of Enforcement Discretion from TS 3.7.9 for Palo Verde Nuclear Generating Station, Units 1, 2, and 3, until an Emergency TS change could be processed and approved. On May 14, 1993, the NRC verbally notified the licensee of its intention to exercise discretion not to enforce compliance with TS 3.7.9 until the licensee requested, and the staff completed its review of, a change to the TS to address the nonconforming condition. This verbal notification was confirmed in a letter dated May 18, 1993.

In a May 20, 1993, letter, the licensee requested an emergency change to the Technical Specification to state that the methodology used for selecting the test sample for snubber functional testing during the current outage for Unit 2 and the previous outage for both Units 1 and 3 was not in accordance with the requirements of Specification 4.7.9 but adequately demonstrated the operability of the snubber population.

The licensee stated that during the most recent test periods in Unit 1, 2, and 3 the number of hydraulic and mechanical snubbers tested would have met the sampling requirements of Sample Plan 1, with the exception of random sampling, for all types of snubbers except the "small" mechanical type in Unit 3. However, based on industry standards, an acceptable confidence level can be achieved utilizing a sample plan which chooses an initial size of ten percent of each type of snubber and for each failure of a given type, the sample is expanded five percent for that type. Therefore, the small mechanical snubbers would have met the sampling requirement for such a plan based on the number of snubbers in the initial snubber population.

Based on the above discussions, the NRC staff finds the change to the TS to allow for continued operation to be acceptable.

3.0 STATEMENT OF EMERGENCY CIRCUMSTANCES

As part of the root cause of failure investigation of the failed snubbers, an evaluation of the snubber testing program was conducted. It was determined that, although testing satisfactorily demonstrated operability of these snubbers, the program was not being conducted in full compliance with TS 3.7.9. On May 12, 1993, PVNGS Unit 1, 2, and 3 entered the ACTION statement for Limiting Condition for Operation 3.7.9. This ACTION statement allows 72 hours to return snubbers to OPERABLE status. At the end of this period, TS 3.0.3 would have had to be entered and plant shutdowns for both Units 1 and 3 would have had to commence. Unit 2, which is currently in an outage, would have been prohibited from changing modes. The licensee requested a notice of enforcement discretion. The NRC staff exercised discretion not to enforce compliance with TS 3.7.9., pending submission and review of an appropriate change to the TS. The licensee requested the TS change on an emergency basis on May 20, 1993, stating that since the issue was just discovered and adequate time has not existed to process the amendment by normal exigent means. Accordingly, pursuant to 10 CFR 50.91 (a)(5), the Commission has determined that there are emergency circumstances warranting prompt approval by the Commission.

4.0 FINAL DETERMINATION OF NO SIGNIFICANT HAZARDS CONSIDERATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant

hazards consideration if operation of that facility in accordance with the amendment would not:

- (1) Involve a significant increase in the probability or consequences of any accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

The licensee stated that the amendment has been evaluated against the standards in 10 CFR 50.92, and the license amendment request involves no significant hazards considerations based on the following:

- (1) The proposed change documents that the test samples used for snubber functional testing during the last surveillance for each unit were adequate to demonstrate operability of the entire snubber population. Since the actual testing performed demonstrated the operability of the snubbers in the past, and the Technical Specification methodology for selecting test samples in the future is not changing, the proposed license amendment does not involve a significant increase in the probability or consequences of an accident.
- (2) The proposed change does not involve any physical modifications to the plant or changes to methods for operating the plant or equipment. The proposed change involves documenting that past practices for selecting snubber functional test samples were adequate for demonstrating operability of the entire snubber population. Since the actual testing performed demonstrated the operability of the snubbers in the past, and the Technical Specification methodology for selecting test samples in the future is not changing, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.
- (3) The proposed change involves documenting that past practices for selecting snubber functional test samples were adequate for demonstrating operability of the entire snubber population. Since the actual testing performed demonstrated the operability of the snubbers in the past, and the Technical Specification methodology for selecting test samples in the future is not changing, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff agrees that the above standards are satisfied and therefore hereby determines that the amendment request involves no significant hazards consideration.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arizona State official was notified of the proposed issuance of the amendment. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission made a final no significant hazards consideration finding regarding this amendment. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Linh Tran

Date: June 21, 1993