

July 2, 1990

Docket Nos. 50-528, 50-529
and 50-530

DISTRIBUTION
Docket File
NRC & LPDRs
PD5 Reading
JZwolinski
LA
TChan
SPeterson
OGC
EJordan

GHill (12)
Wanda Jones
JCalvo
ACRS (10)
DHagan
GPA/PA
ARM/LFMB
Region V (4)

Mr. William F. Conway
Executive Vice President
Arizona Public Service Company
Post Office Box 52034
Phoenix, Arizona 85072-2034

Dear Mr. Conway:

SUBJECT: ISSUANCE OF AMENDMENT NO.51 TO FACILITY OPERATING LICENSE
NO. NPF-41, AMENDMENT NO.37 TO FACILITY OPERATING LICENSE
NO. NPF-51 AND AMENDMENT NO.23 TO FACILITY OPERATING
LICENSE NO. NPF-74 FOR THE PALO VERDE NUCLEAR GENERATING
STATION, UNITS 1, 2, AND 3 (TAC NOS. 74124, 74125 AND 74126)

The Commission has issued the subject amendments, which are enclosed, to the Facility Operating Licenses for Palo Verde Nuclear Generating Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications (Appendix A to each license) in response to your application transmitted by letter dated March 16, 1990 which superseded your July 26, 1989 letter.

The amendments revise Technical Specifications Section 6.5.1 Plant Review Board (PRB) and Section 6.5.2 Technical Review and Control Activities. The amendments reflect an organizational change and change the composition of the PRB in that the specific titles of the members are being deleted and replaced with a generic phrase. The amendments also delete the review of administrative control procedures and changes that do not affect nuclear safety from the PRB responsibilities.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/s/

Sheri R. Peterson, Project Manager
Project Directorate V
Division of Reactor Projects III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.51 to NPF-41
2. Amendment No.37 to NPF-51
3. Amendment No.23 to NPF-74
4. Safety Evaluation

cc: See next page

DRSP/PD5 DRSP/PD5
LLuther SPeterson:dr
5/23/90 5/30/90
[PV1/2/3 AMD 74124]

DRSP/PD5
TLChan
5/31/90

OGC
T6ody
6/5/90

OGC (A)DRSP/D:PD5
6/11/90 JLa...
6/12/90

CP-1

WFO
11

Mr. William F. Conway
Arizona Public Service Company

Palo Verde

cc:

Arthur C. Gehr, Esq.
Snell & Wilmer
3100 Valley Center
Phoenix, Arizona 85073

Mr. Jack R. Newman
Newman & Holtzinger P.C.
1615 L Street, NW, Suite 1000
Washington, DC 20036

Charles R. Kocher, Esq. Assistant
Council
James A. Boeletto, Esq.
Southern California Edison Company
P. O. Box 800
Rosemead, California 91770

Mr. Jack N. Bailey
Vice President Nuclear
Safety and Licensing
Arizona Public Service Company
P.O. Box 52304
Phoenix, Arizona 85072-2034

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
HC-03 Box 293-NR
Buckeye, Arizona 85326

Mr. James M. Levine
Vice President, Nuclear Production
Arizona Public Service Company
P.O. Box 52304
Phoenix, Arizona 85072-2034

Regional Administrator, Region V
U. S. Nuclear Regulatory Commission
1450 Maria Lane
Suite 210
Walnut Creek, California 94596

Mr. Charles B. Brinkman
Washington Nuclear Operations
Combustion Engineering, Inc.
12300 Twinbrook Parkway, Suite 330
Rockville, Maryland 20852

Mr. Charles Tedford, Director
Arizona Radiation Regulatory Agency
4814 South 40 Street
Phoenix, Arizona 85040

Chairman
Maricopa County Board of Supervisors
111 South Third Avenue
Phoenix, Arizona 85003

(10)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51
License No. NPF-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment dated July 26, 1989 as superseded March 16, 1990 by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

9007170354 900702
PDR ADOCK 05000528
P DOC

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 51, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for *Chale M. Trammell*

John T. Larkins, Acting Director
Project Directorate V
Division of Reactor Projects III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 2, 1990

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 51 TO FACILITY OPERATING LICENSE NO. NPF-41

DOCKET NO. STN 50-528

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

6-7

6-8

6-9

InsertPages

6-7

6-8

6-9

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

Senior Reactor Operator (SRO) license requirement for the Operations Manager shall be met if either the Operations Manager or the Operations Supervisor holds a valid SRO license. The holder of the SRO license shall direct the licensed activities of the licensed operators.

6.4 TRAINING

6.4.1 A training program for the unit staff shall be maintained under the direction of the Director, Site Services or his designee and shall meet or exceed the requirements and recommendations of Section 5.0 of ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The Plant Review Board shall function to advise the Plant Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of nine members from the Palo Verde management staff. These positions will be designated by the Plant Director in Administrative Procedures.

The Plant Director shall designate the Chairman and Vice-Chairmen in writing. The Chairman and Vice-Chairmen may be from outside the members provided that they meet ANSI Standard 3.1, 1978.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman, Vice-Chairmen, or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, Vice-Chairmen, or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The PRB shall be responsible for:

- a. Review of all proposed changes to Appendix "A" Technical Specifications.
- b. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Nuclear Safety Group (NSG).
- c. Review of REPORTABLE EVENTS.
- d. Review of unit operations to detect potential nuclear safety hazards.
- e. Performance of special reviews, investigations or analyses and reports thereon as requested by the Vice President-Nuclear Production or PRB Chairman.
- f. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PRB meeting.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. above constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President-Nuclear Production, Plant Director and NSG of disagreement between the PRB and the Plant Director, however, the Plant Director shall have responsibility for resolution of such disagreements.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President-Nuclear, Vice President-Nuclear Production, Plant Director and NSG.

ADMINISTRATIVE CONTROLS

6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.1 The Director, Standards and Technical Support shall assure that each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.

6.5.2.2 Phase I - IV tests described in the FSAR that are performed by the plant operations staff shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager as previously designated by the Vice President-Nuclear Production. Test results shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager.

6.5.2.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Director; or by the Plant Manager; or by the Director, Standards and Technical Support as previously designated by the Vice President-Nuclear Production.

6.5.2.4 Individuals responsible for reviews performed in accordance with 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the supervisory staff, previously designated by the Director Standards and Technical Support to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Plant Director.

6.5.2.6 The station security program and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.

6.5.2.7 The station emergency plan and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.

6.5.2.8 The Director, Standards and Technical Support shall assure the performance of a review by a qualified individual/organization of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering the evaluation, recommendations and disposition of the corrective action to prevent recurrence.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment dated July 26, 1989 as superseded March 16, 1990 by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Part I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.37, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for *Charles M. Trammell*
John T. Larkins, Acting Director
Project Directorate V
Division of Reactor Projects III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 2, 1990

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NO. STN 50-529

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages

6-7

6-8

6-9

Insert Pages

6-7

6-8

6-9

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

Senior Reactor Operator (SRO) license requirement for the Operations Manager shall be met if either the Operations Manager or the Operations Supervisor holds a valid SRO license. The holder of the SRO license shall direct the licensed activities of the licensed operators.

6.4 TRAINING

6.4.1 A training program for the unit staff shall be maintained under the direction of the Director, Site Services or his designee and shall meet or exceed the requirements and recommendations of Section 5.0 of ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The Plant Review Board shall function to advise the Plant Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of nine members from the Palo Verde management staff. These positions will be designated by the Plant Director in Administrative Procedures.

The Plant Director shall designate the Chairman and Vice-Chairmen in writing. The Chairman and Vice-Chairmen may be from outside the members provided that they meet ANSI Standard 3.1, 1978.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman, Vice-Chairmen, or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, Vice-Chairmen, or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The PRB shall be responsible for:

- a. Review of all proposed changes to Appendix "A" Technical Specifications.
- b. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Nuclear Safety Group (NSG).
- c. Review of REPORTABLE EVENTS.
- d. Review of unit operations to detect potential nuclear safety hazards.
- e. Performance of special reviews, investigations or analyses and reports thereon as requested by the Vice President-Nuclear Production or PRB Chairman.
- f. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PRB meeting.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. above constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President-Nuclear Production, Plant Director and NSG of disagreement between the PRB and the Plant Director; however, the Plant Director shall have responsibility for resolution of such disagreements.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President-Nuclear, Vice President-Nuclear Production, Plant Director, and NSG.

ADMINISTRATIVE CONTROLS

6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.1 The Director, Standards and Technical Support shall assure that each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.

6.5.2.2 Phase I - IV tests described in the FSAR that are performed by the plant operations staff shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager as previously designated by the Vice President-Nuclear Production. Test results shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager.

6.5.2.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Director; or by the Plant Manager; or by the Director, Standards and Technical Support as previously designated by the Vice President-Nuclear Production.

6.5.2.4 Individuals responsible for reviews performed in accordance with 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the supervisory staff, previously designated by the Director Standards and Technical Support to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Plant Director.

6.5.2.6 The station security program and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.

6.5.2.7 The station emergency plan and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.

6.5.2.8 The Director, Standards and Technical Support shall assure the performance of a review by a qualified individual/organization of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering the evaluation, recommendations and disposition of the corrective action to prevent recurrence.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.23
License No. NPF-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated July 26, 1989 as superseded March 16, 1990 by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 23, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John T. Larkins, Acting Director
Project Directorate V
Division of Reactor Projects III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 2, 1990

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 23 TO FACILITY OPERATING LICENSE NO. NPF-74

DOCKET NO. STN 50-530

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages

6-7

6-8

6-9

Insert Pages

6-7

6-8

6-9

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

Senior Reactor Operator (SRO) license requirement for the Operations Manager shall be met if either the Operations Manager or the Operations Supervisor holds a valid SRO license. The holder of the SRO license shall direct the licensed activities of the licensed operators.

6.4 TRAINING

6.4.1 A training program for the unit staff shall be maintained under the direction of the Director, Site Services or his designee and shall meet or exceed the requirements and recommendations of Section 5.0 of ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The Plant Review Board shall function to advise the Plant Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of nine members from the Palo Verde management staff. These positions will be designated by the Plant Director in Administrative Procedures.

The Plant Director shall designate the Chairman and Vice-Chairmen in writing. The Chairman and Vice-Chairmen may be from outside the members provided that they meet ANSI Standard 3.1, 1978.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman, Vice-Chairmen, or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, Vice-Chairmen, or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The PRB shall be responsible for:

- a. Review of all proposed changes to Appendix "A" Technical Specifications.
- b. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Nuclear Safety Group (NSG).
- c. Review of REPORTABLE EVENTS.
- d. Review of unit operations to detect potential nuclear safety hazards.
- e. Performance of special reviews, investigations or analyses and reports thereon as requested by the Vice President-Nuclear Production or PRB chairman.
- f. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PRB meeting.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. above constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President-Nuclear Production, Plant Director and NSG of disagreement between the PRB and the Plant Director; however, the Plant Director shall have responsibility for resolution of such disagreements.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President-Nuclear, Vice President-Nuclear Production, Plant Director and NSG.

ADMINISTRATIVE CONTROLS

6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.1 The Director, Standards and Technical Support shall assure that each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.

6.5.2.2 Phase I - IV tests described in the FSAR that are performed by the plant operations staff shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager as previously designated by the Vice President-Nuclear Production. Test results shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager.

6.5.2.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Director; or by the Plant Manager; or by the Director, Standards and Technical Support as previously designated by the Vice President-Nuclear Production.

6.5.2.4 Individuals responsible for reviews performed in accordance with 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the supervisory staff, previously designated by the Director Standards and Technical Support to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Plant Director.

6.5.2.6 The station security program and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.

6.5.2.7 The station emergency plan and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.

6.5.2.8 The Director, Standards and Technical Support shall assure the performance of a review by a qualified individual/organization of every unplanned on-site release of radioactive material to the environs including the preparation and forwarding of reports covering the evaluation, recommendations and disposition of the corrective action to prevent recurrence.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.51 TO FACILITY OPERATING LICENSE NO. NPF-41,
AMENDMENT NO.37 TO FACILITY OPERATING LICENSE NO. NPF-51
AND AMENDMENT NO.23 TO FACILITY OPERATING LICENSE NO. NPF-74
ARIZONA PUBLIC SERVICE COMPANY, ET AL.
PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3
DOCKET NOS. STN 50-528, STN 50-529, AND STN 50-530

1.0 INTRODUCTION

By letter dated July 26, 1989 as superseded March 16, 1990, the Arizona Public Service Company (APS) on behalf of itself, the Salt River Project Agricultural Improvement and Power District, Southern California Edison Company, El Paso Electric Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), requested changes to the Technical Specifications for the Palo Verde Nuclear Generating Station, Units 1, 2, and 3 (Appendix A to Facility Operating License Nos. NPF-41, NPF-51, and NPF-74, respectively). The proposed changes would revise Technical Specification Section 6.5.1 Plant Review Board (PRB) and Section 6.5.2 Technical Review and Control Activities. The proposed amendments reflect an organizational change and change the composition of the PRB in that specific titles of the members are being deleted and replaced with a generic phrase. The proposed changes also delete the review of administrative control procedures and changes that do not affect nuclear safety from the PRB responsibilities.

2.0 DISCUSSION

The licensee has proposed to change TS 6.5.1.2, Plant Review Board (PRB) composition, by replacing the titles of PRB members with the requirement that the PRB will be composed of nine members from the Palo Verde management staff as designated by the Plant Director in Administrative Procedures. The areas of responsibility of the PRB members will also be located in Administrative Procedures. Changes to these procedures are made in accordance with 10 CFR 50.59. In addition, as part of the composition change, the makeup of the PRB will be from management personnel which would allow decisions to be made and approved at an appropriate level. The staff concludes that the proposed TS 6.5.1.2 is acceptable because it defines the number and qualifications of the PRB members.

9007170355 900702
PDR ADOCK 05000528
P PDC

The licensee has proposed to change TS 6.5.1.6, Plant Review Board Responsibilities, by deleting the review of administrative control procedures and changes that do not affect nuclear safety. The purpose of the PRB is that it shall function to advise the Plant Director on all matters relating to nuclear safety. The proposed amendments will allow the PRB to review in greater detail safety significant items. However, the PRB will continue reviewing administrative control procedures as they relate to nuclear safety as well as procedures which may have an impact on operation of the units. In addition, TS Section 6.5.2.1 states that the Director, Standards and Technical Support shall assure that each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety is prepared by a qualified individual or organization. Each such procedure shall be reviewed by an individual or group other than the one which prepared the procedure, however, they may be from the same organization as the individual which prepared the procedure. For these reasons, the staff concludes that the proposed TS 6.5.1.6 is acceptable.

The proposed change also revises TS 6.5.1.7a to correctly reference 6.5.1.6b in lieu of 6.5.1.6c, based on the deletion of the above requirement to review administrative control procedures and changes. Finally, the proposed amendment corrects individuals' titles based on organizational changes and would also delete the term station nuclear production in TS 6.5.2.4. These are editorial and administrative changes, and therefore, are acceptable.

On the basis of its review, the staff concludes that the licensee has provided an acceptable evaluation of these items. Accordingly, the staff finds the proposed changes to be acceptable.

3.0 CONTACT WITH STATE OFFICIAL

The Arizona Radiation Regulatory Agency has been advised of the proposed determination of no significant hazards consideration with regard to these changes. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting or administrative procedures or requirements. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.