

July 29, 1988

Docket Nos.: STN 50-528, STN 50-529
and STN 50-530

Mr. Donald B. Karner
Executive Vice President
Arizona Nuclear Power Project
Post Office Box 52034
Phoenix Arizona 85072-2034

Dear Mr. Karner:

DISTRIBUTION

Docket File

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Region V (4)
EButcher
GHolahan
DHagan
ARM/LFMB
GPA/PA

SUBJECT: ISSUANCE OF AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. NPF-41, AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-51, AND AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-74, FOR THE PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3, RESPECTIVELY (TAC NOS. 68172, 68173 AND 68174)

The Commission has issued the subject Amendments, which are enclosed, to the Facility Operating Licenses for Palo Verde Nuclear Generating Station, Units 1, 2, and 3. The Amendments consist of a change to the Technical Specifications in response to your application dated May 9, 1988.

The Amendments revise Technical Specification Table 3.6-1 to reduce the maximum actuation time of the containment isolation valves which isolate the containment air radioactivity monitor from 12 seconds to 1 second.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Also enclosed is a corrected Page 3/4 1-5 for the Palo Verde Unit 2 Technical Specifications. It was incorrectly captioned when it was revised in Amendment No. 19, dated May 5, 1988.

Sincerely,

George Knighton for

Michael J. Davis, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 35 to NPF-41
2. Amendment No. 22 to NPF-51
3. Amendment No. 11 to NPF-74
4. Safety Evaluation
5. Page 3/4 1-5 of Unit 2 Tech Specs

cc: See next page.

*See previous concurrence

OFC	:*DRSP/PDV:LA:*DRSP/PDV:PM:*OGC	:DRSP/PDV	:	:
NAME	:JLee	:MDavis:cw	:	:GK
DATE	:7/15/88	:7/13/88	:7/22/88	:7/28/88

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8808090148 880729
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P PDC

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Michael J. Davis, Project Manager
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Division of Reactor Projects - III,
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4. Safety Evaluation
5. Page 3/4 1-5 of Unit 2 Tech Specs

cc: See next page.

*See previous concurrence

OFC	:* <td>:DRSP/PDV</td> <td>:</td> <td>:</td> <td>:</td>	:DRSP/PDV	:	:	:
NAME	:JLee :MDavis:cw :	:GKNighton	:	:	:
DATE	:7/15/88 :7/13/88 :7/22/88	:7/29/88	:	:	:

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George Knighton for

Michael J. Davis, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

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cc: See next page.

*See previous concurrence

C	: *DRSP/PDV:LA: *DRSP/PDV:PM: *OGC	: DRSP/PDV :	:	:
ME	: JLee : MDavis: cw :	: GKNighton :	:	:
TE	: 7/15/88 : 7/13/88 : 7/22/88	: 7/28/88 :	:	:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 29, 1988

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Mr. Donald B. Karner
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Sincerely,

for Michael J. Davis, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

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5. Page 3/4 1-5 of Unit 2 Tech Specs

cc: See next page.

Mr. Donald B. Karner
Arizona Nuclear Power Project

Palo Verde

cc:

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Phoenix, Arizona 85073

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U. S. Nuclear Regulatory Commission
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Washington, DC 20009

Mr. Charles B. Brinkman, Manager
Washington Nuclear Operations
Combustion Engineering, Inc.
7910 Woodmont Avenue Suite 1310
Bethesda, Maryland 20814

Arizona Nuclear Power Project

- 2 -

Palo Verde

cc:

Chairman
Arizona Corporation Commission
Post Office Box 6019
Phoenix, Arizona 85003

Arizona Radiation Regulatory Agency
ATTN: Ms. Clara Palovic, Librarian
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Phoenix, Arizona 85040

Mr. Charles Tedford, Director
Arizona Radiation Regulatory Agency
4814 South 40 Street
Phoenix, Arizona 85040

Chairman
Maricopa County Board of Supervisors
111 South Third Avenue
Phoenix, Arizona 85003



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. NPF-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated May 9, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

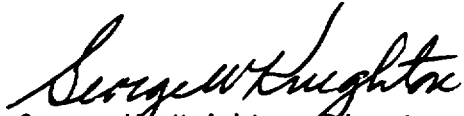
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P PDC

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 35, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

FOR THE NUCLEAR REGULATORY COMMISSION



George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1988

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. NPF-41

DOCKET NO. STN 50-528

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Also to be replaced is the following overleaf page to the amended page.

Amendment Page

3/4 6-21

Overleaf Page

3/4 6-22

TABLE 3.6-1
CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	FUNCTION	MAXIMUM ACTUATION TIME (SECONDS)
A. CONTAINMENT ISOLATION (CIAS)			
RDA-UV 023	9	Containment radwaste sump pump to LRS holdup tank	30
RDB-UV 024	9	Containment radwaste sump pump to LRS holdup tank	5
RDB-UV 407	9	Containment radwaste sump post- accident sampling system	5
SGB-HV 200 [#]	11	Downcomer feedwater chemical injection	1
SGB-HV 201 [#]	12	Downcomer feedwater chemical injection	1
SIA-UV 708 [#]	23	Containment recirc sump to post- accident sampling system	5
HCB-UV 044	25A	Containment air radioactivity monitor (inlet)	1
HCA-UV 045	25A	Containment air radioactivity monitor (inlet)	1
HCA-UV 046	25B	Containment air radioactivity monitor (outlet)	1
HCB-UV 047	25B	Containment air radioactivity monitor (outlet)	1
GAA-UV 002	29	N ₂ to steam generator and reactor drain tank	10
GAA-UV 001	30	N ₂ to SI tanks	10

[#]Not Type C Tested

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	FUNCTION	MAXIMUM ACTUATION TIME (SECONDS)
A. CONTAINMENT ISOLATION (CIAS) (Continued)			
HPA-UV 001	35	Containment to hydrogen recombiner	12
HPA-UV 003	35	Containment to hydrogen recombiner	12
HPA-UV 024	35	H ₂ control system	5
HPB-UV 002	36	Containment to hydrogen recombiner	12
HPA-UV 005	38	Containment to hydrogen recombiner	12
HPB-UV 004	36	H ₂ recombiner return to containment (inlet)	12
HPA-UV 023	38	H ₂ control system	5
HPB-UV 006	39	H ₂ recombiner return to containment (inlet)	12
CHA-UV 516	40	Letdown line from RC loop 2B to regenerative heat exchanger and letdown heat exchanger	5
CHB-UV 523	40	Letdown line from RC loop 2B to regenerative heat exchanger and letdown heat exchanger	5
CHB-UV 924	40	Letdown line to post-accident sampling system	5
SSB-UV 201	42A	Pressurizer liquid sample line	5
SSA-UV 204	42A	Pressurizer liquid sample line	5
SSB-UV 202	42B	Pressurizer steam space sample line	5
SSA-UV 205	42B	Pressurizer steam space sample line	5
SSB-UV 200	42C	Hot leg sample line	5
SSA-UV 203	42C	Hot leg sample line	5



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated May 9, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 22, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

FOR THE NUCLEAR REGULATORY COMMISSION



George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1988

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NO. STN 50-529

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Also to be replaced is the following overleaf page to the amended page.

Amendment Page

3/4 6-21

Overleaf Page

3/4 6-22

TABLE 3.6-1
CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	FUNCTION	MAXIMUM ACTUATION TIME (SECONDS)
A. CONTAINMENT ISOLATION (CIAS)			
RDA-UV 023	9	Containment radwaste sump pump to LRS holdup tank	30
RDB-UV 024	9	Containment radwaste sump pump to LRS holdup tank	5
RDB-UV 407	9	Containment radwaste sump post- accident sampling system	5
SGB-HV 200 [#]	11	Downcomer feedwater chemical injection	1
SGB-HV 201 [#]	12	Downcomer feedwater chemical injection	1
SIA-UV 708 [#]	23	Containment recirc sump to post- accident sampling system	5
HCB-UV 044	25A	Containment air radioactivity monitor (inlet)	1
HCA-UV 045	25A	Containment air radioactivity monitor (inlet)	1
HCA-UV 046	25B	Containment air radioactivity monitor (outlet)	1
HCB-UV 047	25B	Containment air radioactivity monitor (outlet)	1
GAA-UV 002	29	N ₂ to steam generator and reactor drain tank	10
GAA-UV 001	30	N ₂ to SI tanks	10

[#]Not Type C tested.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	FUNCTION	MAXIMUM ACTUATION TIME (SECONDS)
A. CONTAINMENT ISOLATION (CIAS) (Continued)			
HPA-UV 001	35	Containment to hydrogen recombiner	12
HPA-UV 003	35	Containment to hydrogen recombiner	12
HPA-UV 024	35	H ₂ control system	5
HPB-UV 002	36	Containment to hydrogen recombiner	12
HPA-UV 005	38	Containment to hydrogen recombiner	12
HPB-UV 004	36	H ₂ recombiner return to containment (inlet)	12
HPA-UV 023	38	H ₂ control system	5
HPB-UV 006	39	H ₂ recombiner return to containment (inlet)	12
CHA-UV 516	40	Letdown line from RC loop 2B to regenerative heat exchanger and letdown heat exchanger	5
CHB-UV 523	40	Letdown line from RC loop 2B to regenerative heat exchanger and letdown heat exchanger	5
CHB-UV 924	40	Letdown line to post-accident sampling system	5
SSB-UV 201	42A	Pressurizer liquid sample line	5
SSA-UV 204	42A	Pressurizer liquid sample line	5
SSB-UV 202	42B	Pressurizer steam space sample line	5
SSA-UV 205	42B	Pressurizer steam space sample line	5
SSB-UV 200	42C	Hot leg sample line	5
SSA-UV 203	42C	Hot leg sample line	5



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NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. NPF-74


1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated May 9, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 11, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

FOR THE NUCLEAR REGULATORY COMMISSION


George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1988

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-74

DOCKET NO. STN 50-530

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Also to be replaced is the following overleaf page to the amended page.

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3/4 6-21

Overleaf Page

3/4 6-22

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TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

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HPB-UV 002	36	Containment to hydrogen recombiner	12
HPA-UV 005	38	Containment to hydrogen recombiner	12
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SSA-UV 205	42B	Pressurizer steam space sample line	5
SSB-UV 200	42C	Hot leg sample line	5
SSA-UV 203	42C	Hot leg sample line	5



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. NPF-41,
AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-51
AND AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-74
ARIZONA PUBLIC SERVICE COMPANY, ET AL.
PALO VERDE NUCLEAR GENERATING STATION, UNIT NOS. 1, 2 AND 3
DOCKET NOS. STN 50-528, STN 50-529 AND STN 50-530

1.0 INTRODUCTION

By letter dated May 9, 1988, the Arizona Public Service Company (APS) on behalf of itself, the Salt River Project Agricultural Improvement and Power District, Southern California Edison Company, El Paso Electric Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), requested a change to the Technical Specifications for the Palo Verde Nuclear Generating Station, Units 1, 2, and 3 (Appendix A to Facility Operating License Nos. NPF-41, NPF-51 and NPF-74, respectively). The proposed change would revise Technical Specification Table 3.6-1 to reduce the maximum actuation time of the containment isolation valves which isolate the containment air radioactivity monitor from 12 seconds to 1 second.

2.0 EVALUATION

Technical Specification Table 3.6-1 lists containment isolation valves and their maximum actuation times. The operability of the containment automatic isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment. Containment isolation within the time limits specified for those isolation valves designed to close automatically ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for a LOCA.

The valves affected by this change, the isolation valves for the containment air radioactivity monitor, have actual actuation times of less than 1 second. By allowing 12 seconds for valve closure the containment air radioactivity monitor could possibly be subjected to pressures in excess of its 10 psi design limit. FSAR Figure 6.2.1-2, displays containment pressure and temperature response to a double-ended discharge leg slot break, and shows that pressures in excess of 40 psig are possible within 12 seconds of such an event.

The proposed amendments would reduce the allowable actuation times for these valves to 1 second, thereby preserving containment integrity by reducing the possibility of damage to the radiation monitor. This will ensure that the release of radioactive material from the containment atmosphere during accident conditions will be restricted to those leakage paths and associated leak rates assumed in the safety analyses, thereby limiting the site boundary radiation doses to within the 10 CFR Part 100 limits.

On the basis of the above evaluation, the staff concludes that the proposed change to Technical Specification Table 3.6-1 is acceptable.

3.0 CONTACT WITH STATE OFFICIAL

The Arizona Radiation Regulatory Agency was advised of the proposed determination of no significant hazards consideration with regard to this change. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATIONS

The amendments involve changes in the use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

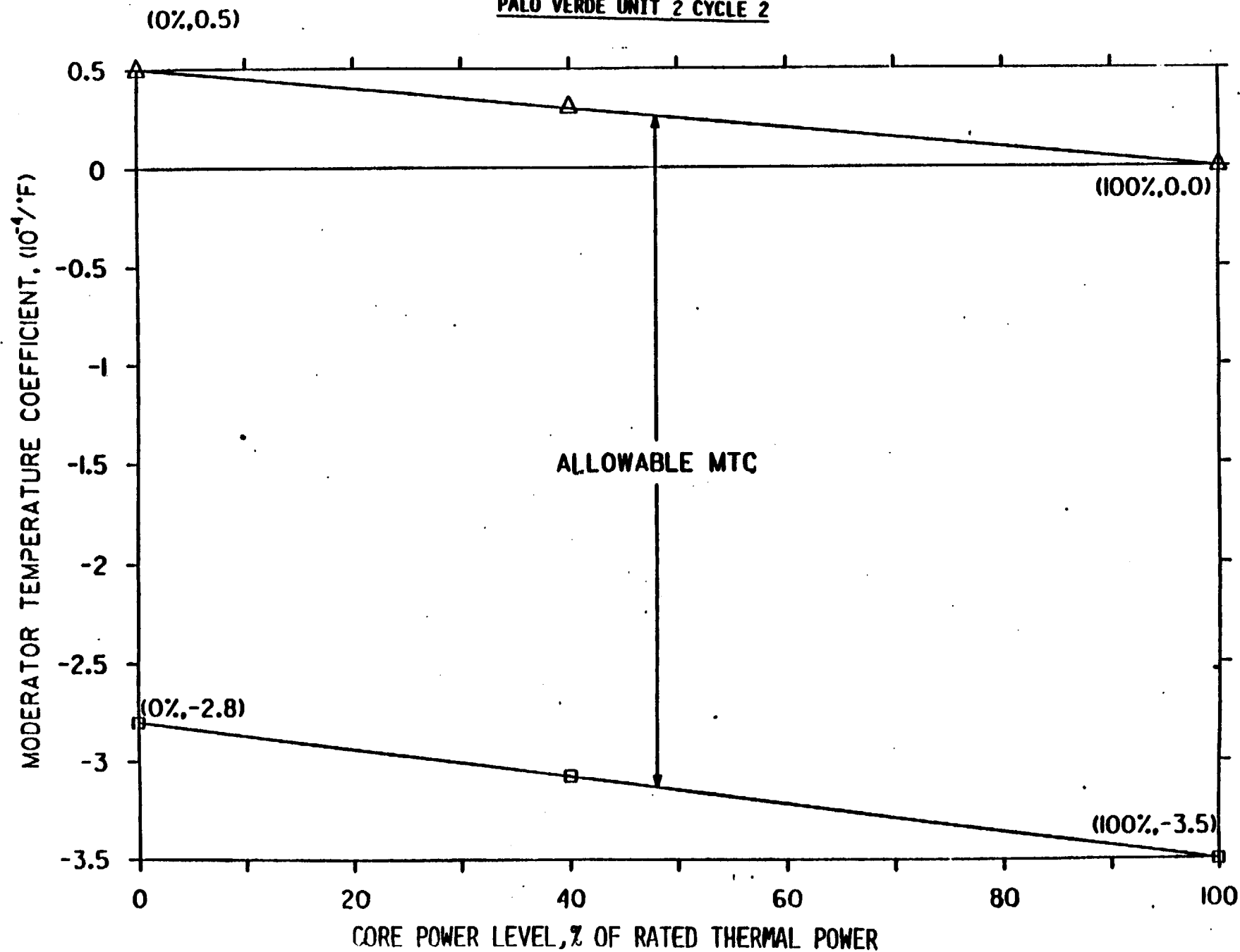
5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public. We therefore, conclude that the proposed change is acceptable.

Principal Contributor: M. Davis

Dated: July 29, 1988

FIGURE 3.1-1
ALLOWABLE MTC MODES 1 AND 2
PALO VERDE UNIT 2 CYCLE 2



MINIMUM TEMPERATURE FOR CRITICALITY

LIMITING CONDITION FOR OPERATION

3.1.1.4 The Reactor Coolant System lowest operating loop temperature (T_{cold}) shall be greater than or equal to 552°F.

APPLICABILITY: MODES 1 and 2#*.

ACTION:

With a Reactor Coolant System operating loop temperature (T_{cold}) less than 552°F, restore T_{cold} to within its limit within 15 minutes or be in HOT STANDBY within the next 15 minutes.

SURVEILLANCE REQUIREMENTS

4.1.1.4 The Reactor Coolant System temperature (T_{cold}) shall be determined to be greater than or equal to 552°F:

- a. Within 15 minutes prior to achieving reactor criticality, and
- b. At least once per 30 minutes when the reactor is critical and the Reactor Coolant System T_{cold} is less than 557°F.

#With K_{eff} greater than or equal to 1.0.

*See Special Test Exception 3.10.5.