

Docket Nos.: 50-528
and 50-529

AUG 11 1986

Mr. E. E. Van Brunt, Jr.
Executive Vice President
Arizona Nuclear Power Project
Post Office Box 52034
Phoenix, Arizona 85072-2034

Dear Mr. Van Brunt:

Subject: Issuance of Amendment No. 7 to Facility Operating License NPF-41
and Amendment No. 1 to Facility Operating License NPF-51
Palo Verde Nuclear Generating Station, Units 1 and 2

The Commission has issued the enclosed Amendment No. 7 to Facility Operating License No. NPF-41 and Amendment No. 1 to Facility Operating License NPF-51 for Palo Verde Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated August 3, 1986 and confirms the telephone notification given to you on August 5, 1986, that the requested change has been granted.

The amendments revise Technical Specification 3/4.7.7 to allow the use of the 1980 version of ANSI Standard N509 in lieu of the 1976 version to meet the guidelines of Regulatory Guide 1.52, Revision 2.

A copy of the Safety Evaluation supporting the amendments is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's Bi-Weekly Notice.

Sincerely,

ISI

E. A. Licitra, Senior Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosures:

1. Amendment No. 7 to NPF-41
2. Amendment No. 1 to NPF-51
3. Safety Evaluation

cc: See next page

wd 8-11-86

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8/6/86

PD7 *JL*
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8/6/86

OGC
[Signature]
8/7/86

PRAD *[Signature]*
DCMutchfield
8/11/86

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GWNighton
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PDR ADOCK 05000528
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Phoenix, Arizona 85007

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Mr. E. E. Van Brunt, Jr.
Arizona Nuclear Power Project

Palo Verde

cc:

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Regional Administrator, Region V
U. S. Nuclear Regulatory Commission
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AUG 11 1986

ISSUANCE OF AMENDMENT NO. 7 TO FACILITY OPERATING
LICENSE NPF-41 FOR PALO VERDE UNIT 1
AND AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NPF-51
FOR PALO VERDE UNIT 2

~~DISTRIBUTION~~

~~Docket File 50-528/529~~

NRC PDR

Local PDR

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7
License No. NPF-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated August 3, 1986, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of the Facility Operating License No. NPF-41 is hereby amended to read as follows:

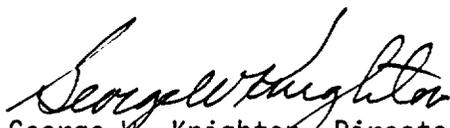
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 7, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of August 5, 1986.

FOR THE NUCLEAR REGULATORY COMMISSION


George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosure:
Change to the Technical
Specifications

Date of Issuance: **AUG 11 1986**

AUG 11 1986

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ENCLOSURE TO LICENSE AMENDMENT NO. 7

FACILITY OPERATING LICENSE NO. NPF-41

DOCKET NO. STN 50-528

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Pages

3/4 7-17
B 3/4 7-5

Overleaf Pages

3/4 7-18
B 3/4 7-6

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the system flow rate is 28,600 cfm \pm 10%.
 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*.
 3. Verifying a system flow rate of 28,600 cfm \pm 10% during system operation when tested in accordance with ANSI N510-1980.
- c. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*.
- d. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filters, pre-filters, and charcoal adsorber banks is less than 8.4 inches Water Gauge while operating the system at a flow rate of 28,600 cfm \pm 10%.
 2. Verifying that on a Control Room Essential Filtration Actuation Signal and on a SIAS, the system is automatically placed into a filtration mode of operation with flow through the HEPA filters and charcoal adsorber banks.
 3. Verifying that the system maintains the control room at a positive pressure of greater than or equal to 1/8-inch Water Gauge relative to adjacent areas during system operation at a makeup flow rate to the control room of less than or equal to 1000 cfm.
 4. Verifying that the emergency chilled water system will maintain the control room environment at a temperature less than or equal to 80°F for a period of 30 minutes.

*ANSI N509-1980 is applicable for this specification.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- e. After each complete or partial replacement of a HEPA filter bank by verifying that the HEPA filter banks remove greater than or equal to 99% of the DOP when they are tested in-place in accordance with ANSI N510-1980 while operating the system at a flow rate of 28,600 cfm \pm 10%.

- f. After each complete or partial replacement of a charcoal adsorber bank by verifying that the charcoal adsorbers remove greater than or equal to 99.95% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1980 while operating the system at a flow rate of 28,600 cfm \pm 10%.

PLANT SYSTEMS

BASES

3/4.7.7 CONTROL ROOM ESSENTIAL FILTRATION SYSTEM

The OPERABILITY of the control room essential filtration system ensures that the control room will remain habitable for operations personnel during and following all credible accident conditions. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criterion 19 of Appendix A, 10 CFR Part 50.

The use of ANSI Standard N509 (1980) in lieu of ANSI Standard N509 (1976) to meet the guidance of Regulatory Guide 1.52, Revision 2, Positions C.6.a and C.6.b, has been found acceptable as documented in Revision 2 to Section 6.5.1 of the Standard Review Plan (NUREG-0800).

3/4.7.8 ESF PUMP ROOM AIR EXHAUST CLEANUP SYSTEM

The OPERABILITY of the ESF pump room air exhaust cleanup system ensures that radioactive materials leaking from the ECCS equipment within the pump room following a LOCA are filtered prior to reaching the environment. The operation of this system and the resultant effect on offsite dosage calculations was assumed in the safety analyses.

3/4.7.9 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety-related systems is maintained during and following a seismic or other event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

Snubbers are classified and grouped by design and manufacturer but not by size. For example, mechanical snubbers utilizing the same design features of the 2-kip, 10-kip, and 100-kip capacity manufactured by Company "A" are of the same type. The same design mechanical snubbers manufactured by Company "B" for the purposes of this Technical Specification would be of a different type, as would hydraulic snubbers from either manufacturer.

A list of individual snubbers with detailed information of snubber location and size and of system affected shall be available at the plant in accordance with Section 50.71(c) of 10 CFR Part 50. The accessibility of each snubber shall be determined and approved by the Plant Review Board. The determination shall be based upon the existing radiation levels and the expected time to perform a visual inspection in each snubber location as well as other factors associated with accessibility during plant operations (e.g., temperature, atmosphere, location, etc.), and the recommendations of Regulatory Guides 8.8 and 8.10. The addition or deletion of any hydraulic or mechanical snubber shall be made in accordance with Section 50.59 of 10 CFR Part 50.

The visual inspection frequency is based upon maintaining a constant level of snubber protection. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number

PLANT SYSTEMS

BASES

SNUBBERS (Continued)

of inoperable snubbers found during an inspection. In order to establish the inspection frequency for each type of snubber, it was assumed that the frequency of failures and initiating events is constant with time and that the failure of any snubber could cause that system to be unprotected and to result in failure during an assumed initiating event. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

The acceptance criteria are to be used in the visual inspection to determine OPERABILITY of the snubbers.

To provide assurance of snubber functional reliability one of three functional testing methods are used with the stated acceptance criteria:

1. Functionally test 10% of a type of snubber with an additional 10% tested for each functional testing failure; or
2. Functionally test a sample size and determine sample acceptance or rejection using Figure 4.7-1, or
3. Functionally test a representative sample size and determine sample acceptance or rejection using the stated equation.

Figure 4.7-1 was developed using "Wald's Sequential Probability Ratio Plan" as described in "Quality Control and Industrial Statistics" by Acheson J. Duncan.

Permanent or other exemptions from the surveillance program for individual snubbers may be granted by the Commission if a justifiable basis for exemption is presented and, if applicable, snubber life destructive testing was performed to qualify the snubbers for the applicable design conditions at either the completion of their fabrication or at a subsequent date. Snubbers so exempted shall be listed in the list of individual snubbers indicating the extent of the exemptions.

The service life of a snubber is established via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc.). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated August 3, 1986, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of the Facility Operating License No. NPF-51 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of August 5, 1986.

FOR THE NUCLEAR REGULATORY COMMISSION


George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosure:
Change to the Technical
Specifications

Date of Issuance: AUG 11 1986

AUG 11 1986

- 3 -

ENCLOSURE TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NO. STN 50-529

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Pages

3/4 7-17
B 3/4 7-5

Overleaf Pages

3/4 7-18
B 3/4 7-6

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the system flow rate is 28,600 cfm \pm 10%.
 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*.
 3. Verifying a system flow rate of 28,600 cfm \pm 10% during system operation when tested in accordance with ANSI N510-1980.
- c. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*.
- d. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filters, pre-filters, and charcoal adsorber banks is less than 8.4 inches Water Gauge while operating the system at a flow rate of 28,600 cfm \pm 10%.
 2. Verifying that on a Control Room Essential Filtration Actuation Signal and on a SIAS, the system is automatically placed into a filtration mode of operation with flow through the HEPA filters and charcoal adsorber banks.
 3. Verifying that the system maintains the control room at a positive pressure of greater than or equal to 1/8-inch Water Gauge relative to adjacent areas during system operation at a makeup flow rate to the control room of less than or equal to 1000 cfm.
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*ANSI N509-1980 is applicable for this specification.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- e. After each complete or partial replacement of a HEPA filter bank by verifying that the HEPA filter banks remove greater than or equal to 99% of the DOP when they are tested in-place in accordance with ANSI N510-1980 while operating the system at a flow rate of 28,600 cfm \pm 10%.

- f. After each complete or partial replacement of a charcoal adsorber bank by verifying that the charcoal adsorbers remove greater than or equal to 99.0% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1980 while operating the system at a flow rate of 28,600 cfm \pm 10%.

PLANT SYSTEMS

BASES

3/4.7.7 CONTROL ROOM ESSENTIAL FILTRATION SYSTEM

The OPERABILITY of the control room essential filtration system ensures that the control room will remain habitable for operations personnel during and following all credible accident conditions. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criterion 19 of Appendix A, 10 CFR Part 50.

The use of ANSI Standard N509 (1980) in lieu of ANSI Standard N509 (1976) to meet the guidance of Regulatory Guide 1.52, Revision 2, Positions C.6.a and C.6.b, has been found acceptable as documented in Revision 2 to Section 6.5.1 of the Standard Review Plan (NUREG-0800).

3/4.7.8 ESF PUMP ROOM AIR EXHAUST CLEANUP SYSTEM

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3/4.7.9 SNUBBERS

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Snubbers are classified and grouped by design and manufacturer but not by size. For example, mechanical snubbers utilizing the same design features of the 2-kip, 10-kip, and 100-kip capacity manufactured by Company "A" are of the same type. The same design mechanical snubbers manufactured by Company "B" for the purposes of this Technical Specification would be of a different type, as would hydraulic snubbers from either manufacturer.

A list of individual snubbers with detailed information of snubber location and size and of system affected shall be available at the plant in accordance with Section 50.71(c) of 10 CFR Part 50. The accessibility of each snubber shall be determined and approved by the Plant Review Board. The determination shall be based upon the existing radiation levels and the expected time to perform a visual inspection in each snubber location as well as other factors associated with accessibility during plant operations (e.g., temperature, atmosphere, location, etc.), and the recommendations of Regulatory Guides 8.8 and 8.10. The addition or deletion of any hydraulic or mechanical snubber shall be made in accordance with Section 50.59 of 10 CFR Part 50.

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PLANT SYSTEMS

BASES

SNUBBERS (Continued)

of inoperable snubbers found during an inspection. In order to establish the inspection frequency for each type of snubber, it was assumed that the frequency of failures and initiating events is constant with time and that the failure of any snubber could cause that system to be unprotected and to result in failure during an assumed initiating event. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

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3. Functionally test a representative sample size and determine sample acceptance or rejection using the stated equation.

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Permanent or other exemptions from the surveillance program for individual snubbers may be granted by the Commission if a justifiable basis for exemption is presented and, if applicable, snubber life destructive testing was performed to qualify the snubbers for the applicable design conditions at either the completion of their fabrication or at a subsequent date. Snubbers so exempted shall be listed in the list of individual snubbers indicating the extent of the exemptions.

The service life of a snubber is established via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc.). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 7 TO FACILITY OPERATING LICENSE NO. NPF-41
AND AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NO. NPF-51
ARIZONA PUBLIC SERVICE COMPANY, ET AL.
PALO VERDE NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2
DOCKET NOS. STN 50-528 AND STN 50-529

1.0 INTRODUCTION

By letter dated August 3, 1986, the Arizona Public Service Company (APS) on behalf of itself, the Salt River Project Agricultural Improvement and Power District, Southern California Edison Company, El Paso Electric Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), requested an emergency change to the Technical Specifications (Appendix A to Facility Operating Licenses NPF-41 and NPF-51) for the Palo Verde Nuclear Generating Station, Units 1 and 2, respectively. The proposed change would revise Technical Specification 3/4.7.7 in each license to allow the use of the 1980 version of ANSI Standard N509 in lieu of the 1976 version to meet the guidelines of Regulatory Guide 1.52, Revision 2, for the control room essential filtration system.

2.0 DISCUSSION

Palo Verde Unit 1 is currently in commercial operation, Palo Verde Unit 2 has received an operating license and is in power ascension testing, and Unit 3 is in preoperational testing and has not yet received an operating license. The design of the control room essential filtration system is the same for all three units.

During a recent review of documentation for the Palo Verde Unit 3 control room essential filtration system, the licensees determined that the charcoal in the filter beds for the control room essential filtration system did not meet one of the requirements of ANSI Standard N509 (1976); i.e., the removal efficiency for Methyl Iodide at 25°C and 95% relative humidity was slightly below the required value of 99%. Since Technical Specification 3/4.7.7 for Palo Verde Units 1 and 2 require that the charcoal filters for the control room essential filtration system meet the requirements of ANSI Standard N509 (1976), the licensees performed an evaluation of these charcoal filters for Units 1 and 2.

The licensees determined from the above evaluation that the removal efficiency for Methyl Iodide at 25°C and 95% relative humidity for the charcoal filters in both trains of Palo Verde Unit 1 and in one train of Palo Verde 2 was between 98% and 99% and, hence, did not meet ANSI Standard N509 (1976). The licensees also evaluated the charcoal filters against the 1980 version of ANSI N509 and determined that the filters met all of the requirements for the 1980 version (this version allows a 97% removal efficiency for Methyl Iodide at 30°C and 95% relative humidity).

As a result of the above findings, the licensees submitted a request, dated August 3, 1986, for an Emergency Technical Specification change to Specification 3/4.7.7 for both Units 1 and 2 to allow the use of ANSI Standard N509 (1980) for testing the charcoal filters in the control room essential filtration system. In the request, the licensees stated that the staff acceptance of the 1980 version of the standard is documented in Revision 2 to Section 6.5.1 of the Standard Review Plan (NUREG-0800).

3.0 EVALUATION

This proposed change falls into the category of an emergency change since failure of the NRC to take action would result in Palo Verde Unit 1 shutting down and remaining shutdown until the request is granted. In addition, Palo Verde Unit 2 would not be able to resume its power ascension test program until the request is granted, which would significantly delay full power operation.

The need for the proposed action was not determined until August 1, 1986. The staff has reviewed the facts concerning the request and concludes that APS has made a timely submittal, that power operation for Unit 1 and the power ascension program for Unit 2 cannot proceed without NRC action, and that action by the licensees could not preclude this situation.

The action requested by the licensees is to change Specification 3/4.7.7 to permit the use of the 1980 version of ANSI Standard N509, in lieu of the 1976 version, for meeting the guidelines of Regulatory Guide 1.52, Revision 2, Positions C.6.a and C.6.b, for charcoal filters in the control room essential filtration system. The use of the 1980 version of ANSI N509 was previously found acceptable for this purpose as documented in Revision 2 of Section 6.5.1 of the Standard Review Plan (NUREG-0800).

The staff has reviewed the information submitted by the licensees. Based on that review and the above evaluation, the staff finds the licensees' request to be acceptable. Staff approval of the request was granted to APS by phone on August 5, 1986.

4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

Based on the review of the licensees' submittal as described herein, the staff has made a final determination that the licensees' amendment request does not involve a significant hazards consideration since operation of Palo Verde Units 1 and 2 with the requested change would not (1) involve a significant increase in the probability or consequences of an accident previously analyzed since the acceptance criteria in the 1980 version of ANSI Standard N509 are still within the assumptions used in accident analyses, (2) create the possibility of a new or different kind of accident from any accident previously analyzed since no changes are being made to the design or operation of the facilities, and (3) involve a significant reduction in a margin of safety since there are only minor differences between the 1976 and 1980 versions of ANSI Standard N509.

5.0 CONTACT WITH STATE OFFICIAL

The State was informed by telephone on August 5, 1986 of the staff's no significant hazards consideration determination. The State contact had no comments on the this determination.

6.0 ENVIRONMENTAL CONSIDERATIONS

These amendments involve a change in an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has also determined that the amendments involve no significant hazards consideration. The amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

7.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the request is acceptable.

Dated: AUG 11 1986