

December 7, 1990

Docket No. 50-483

Mr. Donald F. Schnell
Senior Vice President - Nuclear
Union Electric Company
Post Office Box 149
St. Louis, Missouri 63166

Dear Mr. Schnell:

DISTRIBUTION

NRC PDR	DHagan
Local PDR	EJordan
PDIII-3 r/f	OGC-WF1
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SUBJECT: AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. NPF-30
(TAC NO. 75506)

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1. This amendment revises the Technical Specifications in response to your application dated November 14, 1989.

The amendment revises Technical Specification section 6.4.1, "Training," deleting references superseded by the May 26, 1987 revision to Title 10 of the Code of Federal Regulations, Part 55.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

original signed by

Anthony T. Gody, Jr., Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 60 to License No. NPF-30
2. Safety Evaluation

cc w/enclosures:
See next page

Office: LA/PDIII-3
Surname: PKreutzer
Date: 9/14/90

PM/PDIII-3
TGody:rc
11/14/90

PD/PDIII-3
JHannon
12/7/90

OGC-WF1
11/16/90

DOCUMENT NAME: 75506 AMD

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Mr. D. F. Schnell
Union Electric Company

Callaway Plant
Unit No. 1

cc:

Dr. J. O. Cermack
CFA Inc.
4 Professional Dr., Suite 110
Gaithersburg, MD 20879

Gerald Charnoff, Esq.
Thomas A. Baxter, Esq.
Shaw, Pittman, Potts & Trowbridge
2300 N Street, N. W.
Washington, D. C. 20037

Mr. T. P. Sharkey
Supervising Engineer,
Site Licensing
Union Electric Company
Post Office Box 620
Fulton, Missouri 65251

U. S. Nuclear Regulatory Commission
Resident Inspectors Office
RR#1
Steedman, Missouri 65077

Mr. Alan C. Passwater, Manager
Licensing and Fuels
Union Electric Company
Post Office Box 149
St. Louis, Missouri 63166

Manager - Electric Department
Missouri Public Service Commission
301 W. High
Post Office Box 360
Jefferson City, Missouri 65102

Regional Administrator
U. S. NRC, Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Mr. Ronald A. Kucera, Deputy Director
Department of Natural Resources
P. O. Box 176
Jefferson City, Missouri 65102

Mr. Bart D. Withers
President and Chief
Executive Officer
Wolf Creek Nuclear Operating
Corporation
P. O. Box 411
Burlington, Kansas 66839

Mr. Dan I. Bolef, President
Kay Drey, Representative
Board of Directors Coalition
for the Environment
St. Louis Region
6267 Delmar Boulevard
University City, Missouri 63130



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

UNION ELECTRIC COMPANY

CALLAWAY PLANT, UNIT 1

DOCKET NO. STN 50-483

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. NPF-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Union Electric Company (UE, the licensee) dated November 14, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-30 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 60, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the license. UE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective immediately to be implemented no later than 30 days from the date of its issuance. The licensee will immediately inform the Commission, in writing, of the implementation date.

FOR THE NUCLEAR REGULATORY COMMISSION

James K. Hall for

John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 7, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 60

OPERATING LICENSE NO. NPF-30

DOCKET NO. 50-483

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Corresponding overleaf pages are provided to maintain document completeness.

REMOVE

6-6
6-7

INSERT

6-6
6-7

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3 or 4	MODE 5 or 6
SS	1	1*
SRO	1	None
RO	2	1
EO	2	1
STA	1**	None

- SS - Shift Supervisor with a Senior Operator license on Unit 1
 SRO - Individual with a Senior Operator license on Unit 1
 RO - Individual with an Operator license on Unit 1
 EO - Equipment Operator
 STA - Shift Technical Advisor

The Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the control room while the unit is in MODE 1, 2, 3, or 4, an individual (other than the Shift Technical Advisor) with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Shift Supervisor from the control room while the unit is in MODE 5 or 6, an individual with a valid Senior Operator or Operator license shall be designated to assume the control room command function.

*One SRO, either Shift Supervisor or Operating Supervisor.

**The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Shift Supervisor or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC.

ADMINISTRATIVE CONTROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine plant operating characteristics, NRC issuances, industry advisories, REPORTABLE EVENTS and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving plant safety to the Manager, Nuclear Safety and Emergency Preparedness and the Manager, Callaway Plant.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers located on site. Each shall have a bachelor's degree in engineering or related science and at least 2 years professional level experience in his field.

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

RECORDS

6.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to the Manager, Nuclear Safety and Emergency Preparedness and the Manager, Callaway Plant.

6.2.4 SHIFT TECHNICAL ADVISOR

The Shift Technical Advisor (STA)** shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to the safe operation of the unit.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 with the following exceptions:

6.3.1.1 Shift Supervisors, Operating Supervisors, Reactor Operators, and Shift Technical Advisors shall meet or exceed the qualifications of ANSI/ANS 3.1-1981 as endorsed by Reg. Guide 1.8, Revision 2, with the same exceptions as contained in the current revision to the Operator Licensing Examiner Standards, NUREG-1021, ES-202.

6.3.1.2 The Superintendent, Health Physics, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, for a Radiation Protection Manager.

*Not responsible for sign-off function.

**The STA position shall be manned in MODES 1, 2, 3 and 4 unless the Shift Supervisor or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC.

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Superintendent, Training.

6.4.2 The training programs for Shift Supervisors, Operating Supervisors, Reactor Operators, and Shift Technical Advisors shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Rev. 2, with the same exceptions as contained in the current revision to the Operator Licensing Examiner Standards, NUREG-1021, ES-202, and 10 CFR Part 55.

6.4.3 All other training programs shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978.

6.4.4 Training shall include familiarization with relevant industry operational experience identified by the ISEG.

6.5 REVIEW AND AUDIT

6.5.1 ON-SITE REVIEW COMMITTEE (ORC)

FUNCTION

6.5.1.1 The ORC shall function to advise the Manager, Callaway Plant on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The ORC shall be composed of the:

Chairman:	Manager, Callaway Plant
Member:	Asst Mgr, Oper and Maintenance
Member:	Asst Mgr, Tech Services
Member:	Manager, Operations Support
Member:	Supt, Health Physics
Member:	Supt, Operations Support, QA
Member:	Manager, Nuclear Engineering
Member:	Asst Mgr, Work Control

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the ORC Chairman to serve on a temporary basis.**

MEETING FREQUENCY

6.5.1.4 The ORC shall meet at least once per calendar month and as convened by the ORC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the ORC necessary for the performance of the ORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members of which no more than two shall be alternates.

**Except for the alternate for the Supt, Operations Support, QA, who is appointed by the Manager, Quality Assurance.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

6.5.1.6 The ORC shall be responsible for:

- a. Review of all Administrative Procedures;
- b. Review of the safety evaluations for (1) procedures, (2) change to procedures, equipment, systems or facilities, and (3) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question;
- c. Review of proposed procedures and changes to procedures, equipment, systems or facilities which may involve an unreviewed safety question as defined in 10 CFR 50.59 or involves a change in Technical Specifications;
- d. Review of proposed test or experiments which may involve an unreviewed safety question as defined in 10 CFR 50.59 or requires a change in Technical Specifications;
- e. Review of proposed changes to Technical Specifications or the Operating License;
- f. Investigation of all violations of the Technical Specifications including the forwarding of reports covering evaluation and recommendations to prevent recurrence to the Senior Vice President-Nuclear and to the NSRB;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Review of all REPORTABLE EVENTS;
- i. Review of the plant Security Plan and shall submit recommended changes to the NSRB;
- j. Review of the Radiological Emergency Response Plan and shall submit recommended changes to the NSRB;
- k. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems;
- l. Review of any accidental, unplanned or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Manager, Callaway Plant and to the NSRB;



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. NPF-30

UNION ELECTRIC COMPANY
CALLAWAY PLANT, UNIT 1
DOCKET NO. STN 50-483

INTRODUCTION

In a letter dated November 14, 1989, the Union Electric Company (UE), proposed to revise Technical Specification (TS) section 6.3, "UNIT STAFF QUALIFICATIONS," and section 6.4, "TRAINING." The proposed TS changes delete references to requirements superseded by issuance of the revised rule, 10 CFR Part 55, Operators' Licenses, which became effective on May 26, 1987.

EVALUATION

The proposed modification to Specification 6.3.1 removes the reference to a superseded letter from the NRC to all licensees dated March 28, 1980, and separates the minimum qualification exceptions to ANSI/ANS 3.1-1978 to new specifications 6.3.1.1 and 6.3.1.2.

Proposed Specification 6.3.1.1 requires Shift Supervisors, Operating Supervisors, Reactor Operators, and Shift Technical Advisors to meet or exceed the qualifications specified in ANSI/ANS 3.1-1981 with the same exceptions as contained in the current revision to the Operator Licensing Examiner Standards, NUREG-1021, ES-202. The proposed requirements are more restrictive than the current TS requirements and more explicitly identify the positions requiring these qualifications. The proposed TS change adequately addresses the requirements in 10 CFR Part 55, and, therefore, is acceptable to the staff.

Proposed Specification 6.3.1.2 merely relocates the qualification requirements for the Superintendent, Health Physics, as defined in the current section 6.3.1. The proposed change is merely administrative and, therefore, is acceptable to the staff.

The proposed modification to Specification 6.4.1 removes the reference to a superseded letter from the NRC to all licensees dated March 28, 1980, separates specific training program requirements to Specifications 6.4.2 and 6.4.3, and relocates the statement on inclusion of relevant industry experience to Specification 6.4.4.

Proposed Specification 6.4.2 requires training programs for Shift Supervisors, Operating Supervisors, Reactor Operators, and Shift Technical Advisors to meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Rev. 2 with the same exceptions as contained in the current revision to the Operator Licensing Examiner Standards, NUREG-1021, ES-202 and 10 CFR Part 55. The proposed requirements are more restrictive than the current TS requirements and more explicitly identify the positions requiring these qualifications. The proposed TS change adequately addresses the requirements in 10 CFR Part 55, and, therefore, is acceptable to the staff.

Proposed Specifications 6.4.3 and 6.4.4 relocate the reference to Section 5 of ANSI/ANS 3.1-1978 for all other training programs and the requirement that training include familiarization with relevant industry operational experience, respectively. The proposed changes are administrative in nature, not changing the intent or function of the previously-approved TSs; therefore, the staff finds the proposed changes acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

ACKNOWLEDGEMENT

Principal Contributor: Anthony T. Gody, Jr.

Dated: December 7, 1990