January 31, 1991

Docket No. 50-483

Mr. Donald F. Schnell
Senior Vice President - Nuclear
Union Electric Company
Post Office Box 149
St. Louis, Missouri 63166

Dear Mr. Schnell:

SUBJECT: CORRECTION TO AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. NPF-30

On November 1, 1990, the Commission issued Amendment No. 58 to Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1. This amendment revised the Technical Specifications and associated Bases which contained cycle-specific core operating parameters in accordance with NRC Generic Letter 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications." The amendment was issued in response to your application dated March 6, 1990, as supplemented.

Two footnotes were omitted from page 3/4 1-20 and there was also a typo on the page. The page has been corrected and a copy is enclosed. Please accept our apologies for any inconvenience these errors may have caused you.

Sincerely,

original signed by

Anthony T. Gody, Jr., Project Manager
Project Directorate III-3
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
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cc w/enclosure:
See next page

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DOCUMENT NAME: CORRECTION TO AMD 58

120135

Mr. D. F. Schnell
Union Electric Company

Callaway Plant
Unit No. 1

cc:

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SHUTDOWN ROD INSERTION LIMIT

LIMITING CONDITION FOR OPERATION

3.1.3.5 All shutdown rods shall be limited in physical insertion as specified in the Core Operating Limits Report (COLR).

APPLICABILITY: MODES 1* and 2*#.

ACTION:

With a maximum of one shutdown rod inserted beyond the insertion limit specified in the COLR, except for surveillance testing pursuant to Specification 4.1.3.1.2, within 1 hour either:

a. Restore the rod to within the insertion limit specified in the COLR, or

b. Declare the rod to be inoperable and apply Specification 3.1.3.1.

SURVEILLANCE REQUIREMENTS

4.1.3.5 Each shutdown rod shall be determined to be within the insertion limit specified in the COLR:

a. Within 15 minutes prior to withdrawal of any rods in Control Bank A, B, C, or D during an approach to reactor criticality, and

b. At least once per 12 hours thereafter.

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* See Special Test Exceptions Specifications 3.10.2 and 3.10.3.

With K_{eff} greater than or equal to 1.

ROD DROP TIME

LIMITING CONDITION FOR OPERATION

3.1.3.4 The individual full-length shutdown and control rod drop time from the fully withdrawn position shall be less than or equal to 2.7 seconds from beginning of decay of stationary gripper coil voltage to dashpot entry with:

a. T_{avg} greater than or equal to 551°F, and

b. All Reactor Coolant pumps operating.

APPLICABILITY: MODES 1 and 2.

ACTION:

a. With the rod drop time of any full-length rod determined to exceed the above limit, restore the rod drop time to within the above limit prior to proceeding to MODE 1 or 2.

b. With the rod drop times within limits but determined with three reactor coolant pumps operating, operation may proceed provided THERMAL POWER is restricted to less than or equal to 66% of RATED THERMAL POWER.

SURVEILLANCE REQUIREMENTS

4.1.3.4 The rod drop time of full-length rods shall be demonstrated through measurement prior to reactor criticality:

a. For all rods following each removal of the reactor vessel head,

b. For specifically affected individual rods following any maintenance on or modification to the Control Rod Drive System which could affect the drop time of those specific rods, and

c. At least once per 18 months.