

July 29, 1987

Docket No. 50-483

Mr. Donald F. Schnell
Vice President - Nuclear
Union Electric Company
Post Office Box 149
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Dear Mr. Schnell:

The Commission has issued the enclosed Amendment No. 26 to Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1. This amendment is in response to your application dated March 27, 1987.

The amendment clarifies Item 6.g of Technical Specification Table 3.3-3 regarding the blocking, during normal plant shutdowns and startups, of auxiliary feedwater start signals which are automatically generated upon the trip of both main feedwater pumps.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

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Thomas W. Alexion, Project Manager
Project Directorate III-3
Division of Reactor Projects

Enclosures:

1. Amendment No. 26 to License No. NPF-30
2. Safety Evaluation

cc w/enclosures:

See next page

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Date: 07/15/87

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Union Electric Company

Callaway Plant
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

UNION ELECTRIC COMPANY

CALLAWAY PLANT, UNIT 1

DOCKET NO. STN 50-483

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26
License No. NPF-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Union Electric Company (the licensee) dated March 27, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-30 is hereby amended to read as follows:

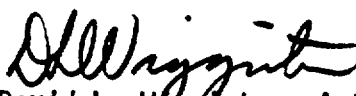
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 26, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the license. UE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David L. Wigginton, Acting Director
Project Directorate III-3
Division of Reactor Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1987

ATTACHMENT TO LICENSE AMENDMENT NO.26

OPERATING LICENSE NO. NPF-30

DOCKET NO. 50-483

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Corresponding overleaf pages are provided to maintain document completeness.

REMOVE

3/4 3-18

3/4 3-20

INSERT

3/4 3-18

3/4 3-20

CALLAWAY - UNIT 1

3/4 3-17

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
5. Feedwater Isolation & Turbine Trip					
a. Automatic Actuation Logic and Actuation Relay (SSPS)	2	1	2	1, 2	27
b. Steam Generator Water Level-High-High	4/stm. gen.	2/stm. gen. in any operating stm gen.	3/stm. gen. in each operating stm. gen.	1, 2	19*
c. Safety Injection	See Item 1. above for all Safety Injection initiating functions and requirements.				
6. Auxiliary Feedwater					
a. Manual Initiation	3(1/pump)	1/pump	1/pump	1, 2, 3	24
b. Automatic Actuation Logic and Actuation Relays (SSPS)	2	1	2	1, 2, 3	21
c. Automatic Actuation Logic and Actuation Relays (BOP ESFAS)	2	1	2	1, 2, 3	21
d. Steam Generator Water Level- Low-Low					
1) Start Motor-Driven Pumps	4/stm. gen.	2/stm. gen. in any operating stm. gen.	3/stm. gen. in each operating stm. gen.	1, 2, 3	19*

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

FUNCTIONAL UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ACTION
8. Loss of Power					
a. 4 kV Bus Undervoltage -Loss of Voltage	4/Bus	2/Bus	3/Bus	1, 2, 3, 4	19*
b. 4 kV Bus Undervoltage -Grid Degraded Voltage	4/Bus	2/Bus	3/Bus	1, 2, 3, 4	19*
9. Control Room Isolation					
a. Manual Initiation	2	1	2	A11	26
b. Automatic Actuation Logic and Actuation Relays (SSPS)	2	1	2	1, 2, 3, 4	26
c. Automatic Actuation Logic and Actuation Relays (BOP)ESFAS)	2	1	2	A11	26
d. Phase "A" Isolation	See Item 3.a above for all Phase "A" Isolation initiating functions and requirements.				
10. Solid-State Load Sequencer	2-1/Train	1/Train	2-1/Train	1, 2, 3, 4	25
11. Engineered Safety Features Actuation System Interlocks					
a. Pressurizer Pressure, P-11	3	2	2	1, 2, 3	20
b. Reactor Trip, P-4	4-2/Train	2/Train	2/Train	1, 2, 3	22

CALLAWAY - UNIT 1

3/4 3-19



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. NPF-30

UNION ELECTRIC COMPANY

CALLAWAY PLANT, UNIT 1

DOCKET NO. STN 50-483

1.0 INTRODUCTION

By letter dated March 27, 1987, Union Electric Company submitted a request to clarify Item 6.g of Technical Specification (TS) Table 3.3-3. The amendment clarifies the blocking, during normal plant shutdowns and startups, of auxiliary feedwater start signals which are automatically generated upon the trip of both main feedwater pumps.

2.0 DISCUSSION

On May 30, 1986, the Technical Specifications (TS) of the Callaway Plant were violated because of an operator error. This resulted in the reactor being operated at power (Modes 1 & 2) with the signal that would have automatically initiated auxiliary feedwater flow in the event of the loss of main feedwater flow being blocked. This violation, which is described in Callaway LER 86-018, attributes improper use of these block switches to inadequate procedural controls and the lack of clarity in the TS as to when the block switches could be employed. The licensee, by letter dated March 27, 1987, proposed to clarify the TS.

The licensee proposed to add a footnote to TS 6.g Table 3.3-3. This would incorporate words similar to those found in Section 7.3.2.7 of the staff's Safety Evaluation Report (SER) for the Callaway Plant (NUREG-0830) i.e.:

"Blocking will be permitted just before shutdown of the last operating main feedwater pump and removed just after the first main feedwater pump is put into service."

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3.0 EVALUATION

During a normal shutdown of the main feedwater pumps, and also during the startup trip test of the main feedwater pumps, the automatic initiation signal for the auxiliary feedwater system should be blocked. This will allow for greater operational flexibility during normal shutdowns and startup of the plant when automatic initiation of auxiliary feedwater flow, upon the trip of both main feedwater pumps, is not needed.

The proposed revision to the TS clarifies the conditions upon which the automatic initiation signal for auxiliary feedwater flow upon a loss of main feedwater flow can be blocked. The proposed wording is consistent with the staff's SER on the Callaway Plant. In addition, the accident analysis in Chapter 15 of the FSAR states that no credit is taken for the signal to initiate auxiliary feedwater when the main feedwater pumps are tripped. Thus, the staff finds the proposed change acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: E. Lantz, SRXB
T. Alexion, PD33

Dated: July 29, 1987