

December 10, 1987

*See correction to Amdt 28
to NPF-30*

Docket No. 50-483

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Mr. Donald F. Schnell
Vice President - Nuclear
Union Electric Company
Post Office Box 149
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Dear Mr. Schnell:

On October 9, 1987, the Commission issued Amendment No. 28 to Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1, in response to your application dated March 31, 1987 as supplemented. The amendment modified the technical specifications to support a transition from a Westinghouse 17 x 17 low-parasitic fuel assembly and optimized fuel assembly fueled core to a Westinghouse 17 x 17 Vantage 5 (V-5) fuel assembly fueled core.

Upon review of the safety evaluation portion of the amendment, it was discovered that the staff inadvertently omitted a discussion on plant effluents during normal operation. This discussion was prepared internally prior to issuance of the amendment, and was part of the evaluation process.

Enclosed is the subject discussion. It should be inserted on page 9 of the original safety evaluation, following the spent fuel pool analysis discussion but prior to the accident analysis (radiological) discussion. We apologize for any inconvenience this may have caused you.

Sincerely,

/s/
Thomas W. Alexion, Project Manager
Project Directorate III-3
Division of Reactor Projects

cc: See next page

Enclosure: As stated

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Union Electric Company

Callaway Plant
Unit No. 1

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PLANT EFFLUENTS DURING NORMAL OPERATION

The licensee provided an assessment of the effect of the extended burnup on the concentrations of significant radionuclides in the reactor coolant system. This assessment considered the effects on normal effluents from the plant including doses to persons in unrestricted areas, and the effects on the classification and processing of solid wastes. Based on this assessment, the licensee concluded that the gaseous and liquid effluents would increase offsite doses by less than 30% and 8%, respectively. These increased levels are well within the 10 CFR Part 50, Appendix I, guidelines. The effects of the classification and processing of solid wastes would be insignificant.

The staff has also reviewed the licensee's assessment of the effect of the extended burnup on the concentrations of significant radionuclides in the reactor coolant system, including the effects on normal effluents from the plant and doses to persons in unrestricted areas in accordance with the criteria of SRP Sections 11.1, 11.2, and 11.3 and the requirements of GDC 60 and 6. Further, the staff reviewed the effects on the classification and processing of solid wastes in accordance with SRP Section 11.4 acceptance criteria and the requirements of GDC 63. The staff finds the licensee's analyses to be acceptable as indicated below. Based on the staff's independent assessment of these effects, the increases in the annual quantities of radioactive materials released in effluents from the plant would be minor and effluent concentrations could be maintained at small fractions of the limits of 10 CFR Part 20. The annual doses to individuals in unrestricted areas can be expected to be maintained below the design objectives of 10 CFR Part 50, Appendix I. The increases in radioactive materials in solid wastes would not be significant and can be processed, transported for disposal, and disposed of in compliance with the requirements of 10 CFR Parts 61 and 71, and any specific requirements of the burial grounds.