

September 21, 1994

50-483

Mr. Donald F. Schnell  
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Union Electric Company  
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SUBJECT: CALLAWAY PLANT, UNIT 1 - CORRECTION TO AMENDMENT NOS. 86 AND 88 TO FACILITY OPERATING LICENSE NO. NPF-30 (TAC NOS. M81213 AND M83006)

Dear Mr. Schnell:

On December 21, 1993, and December 28, 1993, the Commission issued Amendment Nos. 86 and 88, respectively to Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1. These amendments revised the Technical Specifications (TSS) in response to your applications dated August 1, 1991, and February 25, 1992.

Technical Specification pages transmitted with the subject amendments contained typographical errors as follows: pages 3/4 3-26, 3/4 3-52, 3/4 3-53, 3/4 11-2, and 6-2 (Amendment No. 86), and pages 6-10 and 6-13 (Amendment No. 88). These errors have been corrected and the corrected pages are enclosed. Line bars indicate the corrected material.

Please accept our apologies for any inconvenience these administrative errors may have caused.

Sincerely,

Original signed by L. Raynard Wharton

L. Raynard Wharton, Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-483

Enclosure: TS pages 3/4 3-26, 3/4 3-52, 3/4 3-53,  
3/4 11-2, 6-2, 6-10, and 6-13

cc w/encl: See next page

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OFFICE	LA:PDIII-3	PM:PDIII-3	PD:PDIII-3
NAME	MRushbrook	LRWharton:dy	JHannon
DATE	9/20/94	9/20/94	9/21/94

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Callaway Plant  
Unit No. 1

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TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA)</u>	<u>Z</u>	<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
6. Auxiliary Feedwater (Continued)					
d. Steam Generator Water Level Low-Low (Continued)					
2) Start Turbine-Driven Pump (Continued)					
c. Vessel $\Delta T$ Equivalent > 20% RTP					
Coincident with					
Steam Generator Water Level Low-Low (Adverse Containment Environment)	20.2	17.58	2.0	> 20.2% of Narrow Range Instrument Span	> 18.4% of Narrow Range Instrument Span
and					
Containment Pressure - Environmental Allowance Modifier	2.8	0.71	2.0	$\leq$ 1.5 psig	$\leq$ 2.0 psig
OR					
Steam Generator Water Level Low-Low (Normal Containment Environment)	14.8	12.18	2.0	> 14.8% of Narrow Range Instrument Span	> 13.0% of Narrow Range Instrument Span

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CALLAWAY - UNIT 1

3/4 3-25 (F)

Amendment No. 43

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA)</u>	<u>Z</u>	<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
6. Auxiliary Feedwater (Continued)					
e. Safety Injection- Start Motor-Driven Pumps	See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.				
f. Loss-of-Offsite Power- Start Turbine-Driven Pump	N.A.	N.A.	N.A.	N.A.	N.A.
g. Trip of All Main Feedwater Pumps- Start Motor-Driven Pumps	N.A.	N.A.	N.A.	N.A.	N.A.
h. Auxiliary Feedwater Pump Suction Pressure- Low (Transfer to ESW)	N.A.	N.A.	N.A.	≥21.71 psia	≥20.64 psia
7. Automatic Switchover to Containment Sump					
a. Automatic Actuation Logic and Actuation Relays (SSPS)	N.A.	N.A.	N.A.	N.A.	N.A.
b. RWST Level-Low-Low Coincident with Safety Injection	3.4	1.21	2.0	≥36%	≥35.2%
See Item 1. above for Safety Injection Trip Setpoints and Allowable Values.					
8. Loss of Power					
a. 4 kV Undervoltage -Loss of Voltage	N.A	N.A.	N.A.	83V (120V Bus) w/l's delay	83+0,-8.3V (120V Bus) w/l+0.2,-0.5s delay

TABLE 4.3-6

REMOTE SHUTDOWN MONITORING INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. RCS Pressure - Wide Range	M	R
2. Reactor Coolant Temperature - Cold Leg	M	R
3. Source Range, Neutron Flux#	M	R
4. Reactor Trip Breaker Indication	M	N.A.
5. Reactor Coolant Temperature - Hot Leg	M	R
6. Reactor Coolant Pump Breakers	N.A.	N.A.
7. Pressurizer Pressure	M	R
8. Pressurizer Level	M	R
9. Steam Generator Pressure	M	R
10. Steam Generator Level	M	R
11. Auxiliary Feedwater Flow Rate	M	R
12. Auxiliary Feedwater Pump Suction Pressure	M	R

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#Not required OPERABLE in MODE 1 or MODE 2 about P-6 setpoint.

## INSTRUMENTATION

### ACCIDENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

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3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: Modes 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 7 days; otherwise, be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels, except the containment radiation level monitors and the unit vent - high range noble gas monitor, less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 48 hours; otherwise, be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With the number of OPERABLE channels for the containment radiation level monitors or the unit vent - high range noble gas monitor less than the Minimum Channels OPERABLE requirements of Table 3.3-10, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours and either restore the inoperable channel to OPERABLE status within 7 days, or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days that provides actions taken, cause of the inoperability and plans and schedule for restoring the channels to OPERABLE status.
- d. The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure		
a. Normal Range	2	1
b. Extended Range	2	1
2. Reactor Coolant Outlet Temperature - T <sub>HOT</sub> (Wide Range)	2	1
3. Reactor Coolant Inlet Temperature - T <sub>COLD</sub> (Wide Range)	2	1
4. Reactor Coolant Pressure - Wide Range	2	1
5. Pressurizer Water Level	2	1
6. Steam Line Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level - Narrow Range	1/steam generator	1/steam generator
8. Steam Generator Water Level - Wide Range	1/steam generator	1/steam generator
9. Refueling Water Storage Tank Water Level	2	1
10. Containment Hydrogen Concentration Level	2	1
11. Auxiliary Feedwater Flow Rate	1/steam generator	1/steam generator
12. PORV Position Indicator*	1/Valve	1/Valve
13. PORV Block Valve Position Indicator**	1/Valve	1/Valve
14. Safety Valve Position Indicator	1/Valve	1/Valve
15. Containment Normal Sump Water Level	2	1
16. Containment Radiation Level (High Range, GT-RIC-59,-60)	N.A.	1
17. Thermocouple/Core Cooling Detection System	4/core quadrant	2/core quadrant
18. Unit Vent - High Range Noble Gas Monitor (GT-RIC-21B)	N.A.	1

TABLE 3.3-10 (Continued)

TABLE NOTATIONS

\*Not applicable if the associated block valve is in the closed position.

\*\*Not applicable if the block valve is verified in the closed position and power is removed.

## RADIOACTIVE EFFLUENTS

### LIQUID HOLDUP TANKS

#### LIMITING CONDITION FOR OPERATION

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3.11.1.4 The quantity of radioactive material contained in each of the following unprotected outdoor tanks shall be limited to less than or equal to 150 Curies, excluding tritium and dissolved or entrained noble gases:

- a. Reactor Makeup Water Storage Tank,
- b. Refueling Water Storage Tank,
- c. Condensate Storage Tank, and
- d. Outside temporary tanks, excluding demineralizer vessels and the liner being used to solidify radioactive waste.

APPLICABILITY: At all times.

#### ACTION:

- a. With the quantity of radioactive material in any of the above listed tanks exceeding the above limit, immediately suspend all additions of radioactive material to the tank, within 48 hours reduce the tank contents to within the limit, and describe the events leading to this condition in the next Annual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.7.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.11.1.4 The quantity of radioactive material contained in each of the above listed tanks shall be determined to be within the above limit by analyzing a representative sample of the tank's contents at least once per 7 days when radioactive materials are being added and within 7 days following any addition of radioactive material to the tank.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

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3.11.2.5 The concentration of oxygen in the WASTE GAS HOLDUP SYSTEM shall be limited to less than or equal to 3% by volume whenever the hydrogen concentration exceeds 4% by volume.

APPLICABILITY: At all times.

ACTION:

- a. With the concentration of oxygen in the WASTE GAS HOLDUP SYSTEM greater than 3% by volume but less than or equal to 4% by volume, reduce the oxygen concentration to the above limits within 48 hours.
- b. With the concentration of oxygen in the WASTE GAS HOLDUP SYSTEM greater than 4% by volume and the hydrogen concentration greater than 4% by volume, immediately suspend all additions of waste gases to the system and reduce the concentration of oxygen to less than or equal to 4% by volume, then take ACTION a. above.
- c. The provision of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

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4.11.2.5 The concentrations of hydrogen and oxygen in the WASTE GAS HOLDUP SYSTEM shall be determined to be within the above limits by continuously monitoring the waste gases in the WASTE GAS HOLDUP SYSTEM with the hydrogen and oxygen monitors required OPERABLE by Specification 3.3.3.10.

## ADMINISTRATIVE CONTROLS

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### 6.1 RESPONSIBILITY

6.1.1 The Manager, Callaway Plant, shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Supervisor (or during his absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Senior Vice President-Nuclear shall be reissued to all station personnel on an annual basis.

### 6.2 ORGANIZATION

#### 6.2.1 Onsite and Offsite Organization

An onsite and offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organizational charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or equivalent forms of documentation. These requirements shall be documented in the FSAR and updated in accordance with 10 CFR 50.71(e).
- b. The Manager, Callaway Plant shall have responsibility for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Senior Vice President, Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out the health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

## ADMINISTRATIVE CONTROLS

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### 6.2 ORGANIZATION (Continued)

#### UNIT STAFF

6.2.2 The Unit organization shall be subject to the following:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the control room;
- c. An individual from the Health Physics organization#, qualified in radiation protection procedures, shall be on site when fuel is in the reactor;
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least five members# shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Supervisor, and the two other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency;
- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, health physics personnel, equipment operators, and key maintenance personnel.

The amount of overtime worked by Unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12); and

- g. The Superintendent, Operations, Shift Supervisors, and Operating Supervisors shall hold a senior reactor operator license. The Unit Reactor Operator shall hold a reactor operator license or a senior reactor operator license.

#May be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

## ADMINISTRATIVE CONTROLS

### RESPONSIBILITIES (Continued)

- m. Review of Unit operations to detect potential hazards to nuclear safety;
- n. Investigations or analysis of special subjects as requested by the Chairman of the NSRB;
- o. Review of Unit Turbine Overspeed Protection Reliability Program and revisions thereto; and
- p. Review of the Fire Protection Program and submitting recommended changes to the NSRB.

#### 6.5.1.7 The ORC shall:

- a. Recommend in writing to the Manager, Callaway Plant approval or disapproval of items considered under Specifications 6.5.1.6a through e., i., j., k., l., o., and p. above.
- b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6b. through e., and m., above, constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Senior Vice President-Nuclear and the Nuclear Safety Review Board of disagreement between the ORC and the Manager, Callaway Plant; however, the Manager, Callaway Plant shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

### RECORDS

6.5.1.8 The ORC shall maintain written minutes of each ORC meeting that, at a minimum, document the results of all ORC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Senior Vice President-Nuclear and the Nuclear Safety Review Board.

### 6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

#### FUNCTION

6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The NSRB shall report to and advise the Senior Vice President-Nuclear on those areas of responsibility stated in Specifications 6.5.2.8 and 6.5.2.9.

## ADMINISTRATIVE CONTROLS

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### COMPOSITION

6.5.2.2 The NSRB shall be composed of at least the following members:

Chairman: Manager, Licensing and Fuels  
Member: Manager, Nuclear Engineering  
Member: Manager, Nuclear Safety and Emergency Preparedness  
Member: Manager, Quality Assurance  
Member: Vice President, Nuclear Operations  
Member: Supervising Engineer, Nuclear Fuels

Additional members and Vice Chairman may be appointed by the Chairman.

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NSRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NSRB activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSRB Chairman to provide expert advice to the NSRB.

### MEETING FREQUENCY

6.5.2.5 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

### QUALIFICATIONS

6.5.2.6 The NSRB members shall hold a Bachelor's degree in an engineering or physical science field, or equivalent experience, and a minimum of 5 years of technical experience of which a minimum of 3 years shall be in one or more of the disciplines of Specification 6.5.2.1.

### QUORUM

6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least two-thirds of the NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit. For the purpose of a quorum, those considered to have line responsibility will include the Vice President, Nuclear Operations, and personnel reporting to the Vice President, Nuclear Operations.

## ADMINISTRATIVE CONTROLS

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### 6.5.3 TECHNICAL REVIEW AND CONTROL

#### ACTIVITIES

##### 6.5.3.1 Activities which affect nuclear safety shall be conducted as follows:

- a. Procedures required by Specification 6.8 and other procedures which affect plant nuclear safety, and changes thereto, shall be prepared, reviewed and approved. Each such procedure or procedure change shall be reviewed by a qualified individual/group other than the individual/group which prepared the procedure or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Administrative Procedures shall be approved by the appropriate Department Head as designated in writing by the Vice President, Nuclear Operations. The Manager, Callaway Plant, shall approve Administrative Procedures and Radiological Emergency Response Plan implementing procedures. The Manager, Operations Support, shall approve the Security Plan implementing procedures. Temporary changes to procedures which do not change the intent of the approved procedures shall be approved for implementation by two members of the plant staff, at least one of whom holds a Senior Operator license, and documented. The temporary changes shall be approved by the original approval authority within 14 days of implementation. For changes to procedures which may involve a change in intent of the approved procedures, the person authorized above to approve the procedure shall approve the change prior to implementation;
- b. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Manager, Callaway Plant. Each such modification shall be reviewed by a qualified individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Proposed modifications to plant nuclear safety-related structures, systems and components shall be approved prior to implementation by the Manager, Callaway Plant;
- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report or Technical Specifications shall be prepared, reviewed, and approved. Each such test or experiment shall be reviewed by a qualified individual/group other than the individual/group which prepared the proposed test or experiment. Proposed tests and experiments shall be approved before implementation by the Manager, Callaway Plant;

## ADMINISTRATIVE CONTROLS

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### ACTIVITIES (Continued)

- d. Individuals responsible for reviews performed in accordance with Specifications 6.5.3.1a., 6.5.3.1b., and 6.5.3.1c., shall be members of the management staff previously designated by the Manager, Operations Support. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by qualified personnel of the appropriate discipline;
- e. Each review shall include a determination of whether or not an unreviewed safety question is involved. Pursuant to Section 50.59, 10 CFR, NRC approval of items involving unreviewed safety questions shall be obtained prior to the Manager, Callaway Plant, approval for implementation; and
- f. The Plant Security Plan and Radiological Emergency Response Plan, and implementing procedures, shall be reviewed at least once per 12 months. Recommended changes to the implementing procedures shall be approved in accordance with 6.5.3.1.a. Recommended changes to the Plans shall be reviewed pursuant to the requirements of Specifications 6.5.1.6 and 6.5.2.8 and approved by the Manager, Callaway Plant. NRC approval shall be obtained as appropriate.

### RECORDS

6.5.3.2 Records of the above activities shall be provided to the Manager, Callaway Plant, ORC and/or NSRB as necessary for required reviews.

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 of 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the ORC and submitted to the NSRB and the Senior Vice President-Nuclear.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Senior Vice President-Nuclear and the NSRB shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the ORC. This report shall describe: (1) applicable circumstances preceding the violation; (2) effects of the violation upon facility components, systems or structures; and (3) corrective action taken to prevent recurrence;