

June 7, 2002

LICENSEE : Duke Energy Corporation

FACILITIES: McGuire, Units 1 and 2, and Catawba, Units 1 and 2

SUBJECT: TELECOMMUNICATION WITH DUKE ENERGY CORPORATION TO DISCUSS
THE RESPONSE TO A REQUEST FOR ADDITIONAL INFORMATION
PERTAINING TO SECTION 3.5 OF THE LICENSE RENEWAL APPLICATION

On January 28, 2002, the NRC staff (hereafter referred to as "the staff") issued a request for additional information (RAI) pertaining to Section 3.5, Aging Management of Containments, Structures and Component Supports, of the license renewal application (LRA). Duke Energy Corporation (hereafter referred to as "the applicant") responded to this request by letter dated March 11, 2002. On May 28, 2002, a conference call was conducted between the NRC and Duke Energy Corporation to discuss information that was provided to the NRC in response to RAI 3.5-4. Participants of the May 28, 2002, conference call are provided in an attachment.

The staff requested the applicant to expand upon their RAI response by explaining why the bellows (subject to cracking from exposure to chloride) was unique and different from the other components listed in the RAI (fuel transfer canal liner plate, sump liner, and sump screens). The applicant indicated that a leaking bellows had been identified in 1993 and was replaced in 1994. In 1997, leakage from the replacement bellows was identified, and the leaking bellows was replaced. A root cause determination attributed the 1997 bellows leak to transgranular stress-corrosion cracking (TGSCC) as a result of exposure to or contact with chlorine. The applicant could not determine the source of chlorine and speculated that the contaminant could have been introduced by a surface brightener during the manufacturing process. The applicant further stated that TGSCC had not been listed as an applicable aging effect for the other components (fuel transfer canal liner plate, sump liner, and sump screens) because the normal operating environment would not expose these components to chlorine and they essentially consist of plate material that had not been polished or brightened by the manufacturer.

The staff finds the applicant's explanation of why cracking caused by TGSCC was not identified as an applicable aging effect for fuel transfer canal liner plate, sump liner, and sump screens reasonable, but may characterize this as a Confirmatory Item in the Safety Evaluation Report pending the staff's receipt of this information, via letter, to augment the information provided in the applicant's RAI response.

A draft of this telecommunication summary was provided to the applicant to allow them the opportunity to comment prior to the summary being issued.

/RA/

Rani L. Franovich, Project Manager
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-369, 50-370, 50-413, and 50-414

Attachment: As stated

cc w/attachment: See next page

A draft of this telecommunication summary was provided to the applicant to allow them the opportunity to comment prior to the summary being issued.

/RA/

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Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-369, 50-370, 50-413, and 50-414

Attachment: As stated

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