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MCGUIRE NUCLEAR STATION
SELECTED LICENSEE COMMITMENTS
MANUAL (SLC)

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H B BARRON, JR.
VICE PRESIDENT
MCGUIRE NUCLEAR STATION

BY:
P T VU MG01RC PTV/CMK

16.7 INSTRUMENTATION

16.7.2 Seismic Instrumentation

COMMITMENT The seismic monitoring instrumentation shown in Table 16.7.2-1 shall be OPERABLE.

APPLICABILITY: At all times.

REMEDIAL ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more seismic monitoring instruments inoperable.	A.1 Restore inoperable instrument to OPERABLE status. OR A.2 Prepare and submit a Special Report outlining the cause of the malfunction and plans for restoring the instrument(s) to OPERABLE status.	30 days 40 days
B. Seismic monitoring instruments actuated during a seismic event ≥ 0.01g.	B.1 Restore instrument to OPERABLE status. AND B.2 Retrieve data from accessible actuated instruments and analyze to determine magnitude of vibratory ground motion. AND	Within 24 hours following the seismic event Within 24 hours following the seismic event (continued)

REMEDIAL ACTIONS (continued)

B.	(continued)	B.3	Perform Test Record Recording of triaxial time-history accelerograph using internal test and calibrate function.	Prior to removing data
		AND		
		B.4	Perform Test Record Recording of triaxial time-history accelerograph using internal test and calibrate function.	Immediately following insertion of new recording media
		AND		
		B.5	Prepare and submit a Special Report, with copy to Chief, Structural and Geotechnical Engineering Branch, describing the magnitude, frequency spectrum, and resultant effect upon facility features important to safety.	10 days

TESTING REQUIREMENTS

----------NOTE----------
Refer to Table 16.7.2-1 to determine which TRs apply for each Seismic Instrumentation.

	TEST	FREQUENCY
TR 16.7.2.1	----------NOTE---------- CHANNEL CHECK does not include seismic trigger for 1MIMT 5000 or 1MIMT5010. ---------- Perform CHANNEL CHECK.	31 days
TR 16.7.2.2	Perform CHANNEL OPERATIONAL TEST.	6 months
TR 16.7.2.3	Perform a CHANNEL CALIBRATION.	18 months

TABLE 16.7.2-1

SEISMIC MONITORING INSTRUMENTATION

INSTRUMENTS AND SENSOR LOCATIONS		MEASUREMENT RANGE	REQUIRED CHANNELS	TESTING REQUIREMENTS
1.	Triaxial Time-History Accelerographs			
1.a	1MIMT 5000 (Remote Sensor A) Containment Base Slab	0-1g	1	TR 16.7.2.1 TR 16.7.2.2 TR 16.7.2.3
1.b	1MIMT 5010 (Remote Sensor B) Containment Wall Elev. 786' 5"	0-1 g	1	TR 16.7.2.1 TR 16.7.2.2 TR 16.7.2.3
1.c	1MIMT 5020 (Starter Unit) Containment Base Slab	0.005-0.05g	1	TR 16.7.2.2 TR 16.7.2.3
2.	Triaxial Peak Accelerographs			
2.a	1MIMT 5030 Containment Bldg. Elev. 786' 8 9/16"	0-2g	1	TR 16.7.2.3
2.b	1MIMT 5040 Containment Bldg. Elev. 746' 2 1/2"	0-2g	1	TR 16.7.2.3
2.c	1MIMT 5050 Auxiliary Bldg. Elev. 716' 6"	0-2g	1	TR 16.7.2.3
3.	Triaxial Seismic Switches			
	1MIMT 5060 Containment Base Slab	0.025-0.25g	1*	TR 16.7.2.1 TR 16.7.2.2 TR 16.7.2.3
4.	Triaxial Response-Spectrum Recorders			
4.a	1MIMT 5070 Containment Base Slab	0-2g	1*	TR 16.7.2.1 TR 16.7.2.2 TR 16.7.2.3
4.b	1MIMT 5080 Containment Bldg. Elev. 751' 8 1/4"	0-2g	1	TR 16.7.2.3
4.c	1MIMT 5090 Auxiliary Bldg. Elev. 750'	0-2g	1	TR 16.7.2.3

* With reactor control room indication.

BASES

The OPERABILITY of the seismic instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the facility to determine if plant shutdown is required pursuant to Appendix A of 10 CFR Part 100. The instrumentation is consistent with the recommendations of Regulatory Guide 1.12, Instrumentation for Earthquakes, April 1974.

The triaxial time-history accelerograph records seismic data to a magnetic tape. A test record must be initially written to the magnetic tape to arm the system. When seismic event data is captured, another test record must be written to the tape for the purpose of data correlation before the tape is removed. When the magnetic tape is replaced, a test record is written again to the new magnetic tape to rearm the system so it can capture subsequent seismic events. Other passive triaxial accelerographs are also used to gather seismic data to determine the facility response.

REFERENCES

1. Regulatory Guide 1.12, Instrumentation for Earthquakes, April 1974.

2. 10 CFR Part 100, Appendix A.

16.13 CONDUCT OF OPERATIONS

16.13.2 Technical Review and Control

COMMITMENT

A Technical Review and Control Program covering preparation, review, and approval of documents important to station operation shall be established and maintained for the site.

This commitment applies to the review and control activities described in items a through j as listed below. Personnel performing the preparation, review, and approval activities covered by this commitment shall meet or exceed the qualifications of ANSI N18.1-1971. The conformance status for this standard is as listed in Table 17-1 of the Duke Energy Corporation Topical Report, Quality Assurance Program, Duke-I-A (hereafter referred to as the Quality Assurance Program Topical Report).

a. The preparation, review, and approval of station procedures shall be done in accordance with the Quality Assurance Program Topical Report. Individuals responsible for these reviews shall have been previously designated by the Executive Vice President, Nuclear Generation, or direct reports, or the Site Vice President, or direct reports, to perform such reviews and have, as a minimum, a high school diploma or equivalent, and four years technical experience. Review of environmental radiological analysis procedures, shall be performed by the Manager, Environmental Radiological Laboratory or a designee. Each such review shall include a determination of whether or not additional, cross-disciplinary review shall be performed by the appropriately designated site review personnel.

b. Proposed modifications shall be designed and the design reviewed in accordance with the Quality Assurance Program Topical Report. The proposed modification design, the design review, and design approval shall be in accordance with ANSI N45.2.11 as described in Table 17-1 of the Quality Assurance Program Topical Report. Proposed modifications to nuclear safety related structures, systems, and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Maintenance Superintendent, the Operations Superintendent, the Work Control Superintendent, or the on Duty Emergency Coordinator as previously designated by the Station Manager. Upon implementation approval, the modification shall be implemented in accordance with the Duke Power Nuclear Station Modification Program and approved procedures (as discussed in Item a above).

COMMITMENT (continued)

c. Proposed changes to the station Technical Specifications or Facility Operating License shall be prepared in accordance with the Quality Assurance Program Topical Report. Proposed Technical Specification changes (including affected Bases) or Facility Operating License changes shall be reviewed by the Plant Operations Review Committee (PORC) and the Nuclear Safety Review Board (NSRB) prior to submittal to the Nuclear Regulatory Commission. For proposed changes that are determined to be administrative in nature (as defined by Reference 4), these reviews may be performed by only the Chairperson of the PORC and the Director of the NSRB. Proposed changes to the Technical Specifications or Facility Operating License shall be approved by the Station Manager, or for the Station Manager by a designated manager or company officer. Technical Specifications or Facility Operating License submittal cover letters shall be signed by an officer of Duke Power Company.

d. Proposed tests and experiments which affect station nuclear safety and are not addressed in the UFSAR or Technical Specifications shall be reviewed by the Plant Operations Review Committee (PORC).

e. Incidents reportable pursuant to 10 CFR 50.73 requirements and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Manager, Safety Assurance and provided to the Site Vice President and the Plant Operations Review Committee (PORC).

f. The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President. Such reports shall be provided to the Plant Operations Review Committee (PORC).

g. The Manager, Safety Assurance shall assure the performance of a review by a knowledgeable individual/organization of every unplanned onsite release of radioactive material to the environs, including the preparation and forwarding of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence to the Site Vice President, and to the Plant Operations Review Committee (PORC).

h. The Manager, Safety Assurance shall assure the performance of a review by a knowledgeable individual/organization of changes to the Process Control Program, Off site Dose Calculation Manual (ODCM), and radwaste treatment systems.

COMMITMENT (continued)

i. The Manager, Safety Assurance shall ensure the performance of a review by a knowledgeable individual/organization of the Fire Protection program and implementing procedures and submittal of recommended changes to a Nuclear General Office fire protection engineer that is qualified in accordance with the Quality Assurance Program Topical Report.

j. Reports documenting each of the activities performed under this commitment shall be maintained. Copies shall be provided to the NSRB.

APPLICABILITY At all times.

REMEDIAL ACTIONS

None

TESTING REQUIREMENTS

None

BASES

The requirements contained in this selected licensee commitment were relocated from the McGuire Technical Specifications with the approval of the U. S. Nuclear Regulatory Commission. Changes to this SLC shall be considered a change in an NRC commitment and shall be made only in accordance with the approved Nuclear System Directive for the Control of Selected Licensee Commitments and by use of the 10 CFR 50.59 process.

This SLC implements the review requirements of ANSI N18.7-1976/ANS-3.2 and ANSI N45.2.11-1974 as referenced in the Quality Assurance Program Topical Report.

REFERENCES

1. ANSI N18.1-1971, Selection and Training of Nuclear Power Plant Personnel.

2. ANSI N18.7-1976/ANS-3.2, Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants.

3. ANSI N45.2.11-1974, Quality Assurance Requirements for the Design of Nuclear Power Plants.

4. Nuclear System Directive 221, Facility Operating License and Technical Specifications Amendments/Selected Licensee Commitments/Technical Specifications Bases Changes.

5. Nuclear System Directive 209, 10 CFR 50.59 Process.

6. 10 CFR 50.59.

7. Nuclear System Directive 703, Administrative Instructions for Station Procedures.

8. Nuclear System Directive 228, Applicability Determination.

9. Duke Energy Corporation Topical Report, Quality Assurance Program, Duke-I-A.

16.13 CONDUCT OF OPERATIONS

16.13.3 Plant Operations Review Committee

COMMITMENT

A Plant Operations Review Committee (PORC) shall be established and maintained for the site. The PORC shall be composed of the Manager of Safety Assurance, the Station Manager and his/her direct reports most responsible for station operation and maintenance, the Manager of Engineering and his/her direct reports most responsible for engineering support of station operation and maintenance, or designated alternates. The PORC Chairperson, members, and alternate members shall be qualified in accordance with ANSI N18.1-1971 and be appointed by the Site Vice President.

The quorum necessary for conducting the PORC functions shall consist of the Chairperson, or his/her designated alternate, and at least three other PORC members including alternates.

Reports of reviews encompassed by this Selected Licensee Commitment shall be prepared and forwarded to the Site Vice President and the Nuclear Safety Review Board.

The PORC shall be responsible for reviewing the following prior to final approval:

1. All proposed tests and experiments which affect station nuclear safety and are not addressed in the UFSAR or Technical Specifications;

2. Operability evaluations resulting in a Justification for Continued Operation and a proposal for discretionary enforcement;

3. Operability evaluations resulting in the decision that compensatory actions are necessary for affected systems, structures or components to be OPERABLE; and

4. Proposed changes to the station Technical Specifications (and affected Bases that are included with the license amendment request submittal package) or Facility Operating License. For proposed changes that are determined to be administrative in nature (as defined by Reference 4), this review may be performed by only the Chairperson of the PORC.

The PORC shall be responsible for reviewing the effectiveness of corrective actions to prevent recurrence for:

1. Licensee Event Reports and Special Reports made to the NRC;

2. Violations of Technical Specifications;

COMMITMENT (continued)

3. Special reviews and investigations as requested by the Site Vice President; and

4. Reports on unplanned onsite releases of radioactive material to the environs.

The PORC shall review additional programs, procedures and plant activities as directed by the Site Vice President.

APPLICABILITY At all times for those activities listed.

REMEDIAL ACTIONS

None

TESTING REQUIREMENTS

None

BASES

The PORC shall be established to recommend to the Station Manager approval or disapproval of the items listed under APPLICABILITY prior to their final approval.

The PORC shall report to the Site Vice President on the areas of responsibility specified in this selected licensee commitment.

REFERENCES

1. ANSI N18.1-1971, Selection and Training of Nuclear Power Plant Personnel

2. ANSI N18.7-1976/ANS-3.2, Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants

3. Nuclear System Directive 203, Operability

4. Nuclear System Directive 221, Facility Operating License and Technical Specifications Amendments/Selected Licensee Commitments/Technical Specifications Bases Changes

5. Nuclear System Directive 308, Plant Operations Review Committee