

May 26, 1999

Docket
file

Mr. Randal K. Edington
Vice President Operations
Energy Operations, Inc.
River Bend Station
P. O. Box 220
St. Francisville, LA 70775

SUBJECT: CORRECTION TO RIVER BEND STATION AMENDMENT NO. 106

Dear Mr. Edington:

The Nuclear Regulatory Commission (NRC) issued License Amendment No. 106 to Facility Operating License No. NPF-47 to the River Bend Station, Technical Specifications (TSs), on May 5, 1999. The amendment deleted the limits on power and flow conditions in the specifications on recirculation loops operating, added two new specifications to the TSs to establish limits for fraction of core boiling boundary and the period based detection system, modified the surveillance requirements for reactor protection instrumentation, and added NEDO-32339, Revision 1 "Reactor Stability Long-Term Solution, Enhanced Option I-A," document to the core operating limits report.

Due to an administrative oversight, TS pages 3.4-4 and 3.4-5 were inadvertently omitted when issuance of the amendment. These pages should replace the existing pages in the TSs and are enclosed. TS Bases page B 3.3-25 contained typographical errors that did not reflect the marked-up page submitted by the original amendment application. Corrected Bases page B 3.3-25 should replace the existing page and is enclosed. The staff's evaluation and conclusions contained in its Safety Evaluation dated May 5, 1999, are not affected by these corrections.

We regret any inconvenience this may have caused.

Sincerely,

ORIGINAL SIGNED BY

Robert J. Fretz, Project Manager, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-468⁴⁵⁸

Enclosure: As stated

cc w/encl: See next page

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CORRECTION TO AMENDMENT NO. 106
TO FACILITY OPERATING LICENSE NO. NPF-47
RIVER BEND STATION, UNIT 1

Please remove and replace the following Technical Specification pages:

<u>Remove</u>	<u>Insert</u>
3.4-4	3.4-4
3.4-5	3.4-5
B 3.3-25	B 3.3-25

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BASES

SURVEILLANCE
REQUIREMENTS
(continued)

SR 3.3.1.1.2

To ensure that the APRMs are accurately indicating the true core average power, the APRMs are calibrated to the reactor power calculated from a heat balance. The Frequency of once per 7 days is based on minor changes in LPRM sensitivity, which could affect the APRM reading between performances of SR 3.3.1.1.8.

A restriction to satisfying this SR when $< 25\%$ RTP is provided that requires the SR to be met only at $\geq 25\%$ RTP because it is difficult to accurately maintain APRM indication of core THERMAL POWER consistent with a heat balance when $< 25\%$ RTP. At low power levels, a high degree of accuracy is unnecessary because of the large inherent margin to thermal limits (MCPR and APLHGR). At $\geq 25\%$ RTP, the Surveillance is required to have been satisfactorily performed within the last 7 days in accordance with SR 3.0.2. A Note is provided which allows an increase in THERMAL POWER above 25% if the 7 day Frequency is not met per SR 3.0.2. In this event, the SR must be performed within 12 hours after reaching or exceeding 25% RTP. Twelve hours is based on operating experience and in consideration of providing a reasonable time in which to complete the SR.

SR 3.3.1.1.3

The Average Power Range Monitor Flow Biased Simulated Thermal Power High Function uses a trip level generated by the flow control trip reference card based on the recirculation loop drive flow. The drive flow is adjusted by a digital algorithm according to selected drive flow alignment dip switch settings. This SR sets the flow control trip reference card to ensure the drive flow alignment used results in the appropriate trip level being generated from the digital components of the card.

The Frequency of once following a refueling outage is based on the expectation that any change in the core flow to drive flow functional relationship during power operation would be gradual and

(continued)