Oyster Creek Generating Station NRC Operating Examination – May 2002 Validation Changes

Admin JPMs

200.0A d	change tolerance	for correct	t answer from C).1 to 0.2 , step {
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Operational JPMs

201.02	Add cue after step 4, If Reactor Engineer contacted – state that
	under current conditions it is permissible to follow the ABN actions.

- 239.01 Change task conditions, add, Plant is in an ATWS condition, crew is preparing to lower level.
- 202.10 Add second cue after step 1; CUE: the other RO will attend to all alarms not associated with your present task.

NOTE: after step 6, coordinate with/contact electrician directly.

- 226.01 Corrected Task Conditions (ATWS and LOCA) and incorporated ability to secure recirc pumps and drywell fans.
- 264.01 Enhanced initiating cue: Load diesel to 2800 KW and adjust KVARS to 1000 KVARS.

Added CUE after step 3, EO may be sent to EDG building.

Step 11 – change to 2800. Add cue after step 11; change to 1000.

- 261.0N Changed JPM to be consistent with Support Procedure 49
- 279.06 Made step 5 & 6 NOT critical

Scenario changes

SRO#1 Event 5, Add NOTE: Due to a loss of all condensate pumps, an automatic scram on low reactor water level may occur before a manual scram is directed.

Event 8, Changed event description; deleted Manual and added NOTE: EMRV may open automatically due to high reactor pressure. The valve will fail open after either manual or automatic actuation.

SRO#2 Event 2, edit Cause; change shutdown to **trip**.

Event 4, added review of TS 3.5, Containment Integrity

Event 6, add NOTE: The evaluator may direct the leakrate to be increased, if required

SRO#3 Event 2, add NOTE: If the candidate checks the Reactor Building ambient temperature (on the back panels), tell him that the ambient temperature is 75 degrees F.

Event 7, Deleted, no replacement required

SRO#4 Shift Turnover, eliminated Backup EMRV position indication OOS

Event 2, changed TS evaluation description

Event 3, changed power limit to 569.5 Mwe

Event 6, added Support Procedure references

ES-301

Administrative Topics Outline

Form ES-301-1

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Administrative Topic/Subject Description		Describe method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions		
A.1	Temporary Procedure Change JPM	Temporary Procedure Change - Alternate Path (JPM) Change of intent		
	Plant Parameter Verification JPM	Verification of Identified Leakage (JPM)		
A.2	Technical Specification Equipment Control JPM	Tech Spec interpretation and log entry (JPM)		
A.3	Control of Radiation Releases JPM	Review and approve a Liquid Radwaste Discharge – Alternate Path (JPM) Authorize pumping 1-5 sump overboard, SW rad monitor		
A.4	Emergency Classification JPM	Make an Emergency Classification (JPM) Security based		

Control Room Systems and Facility Walk-Through Test Outline ES-301

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B.1 Control Room Systems		•
System / JPM Title	Type Code*	Safety Function
a. Reactor Manual Controls (RMC) / Perform rod coupling check (Alternate Path – Rod Uncoupled)	201.01 M, S, A, L	1
b. Reactor Feedwater System / Place third feedwater pump in service	259.0N N, S	2
c. Main Steam / Bypass a MSIV low-low isolation signal	239.01 D, S	3
d. Recirculation system / Respond to a tripped recirc pump with 5 operating (Alternate Path – Discharge Valve will not close)	202.10 D, S, A	4
e. Containment Spray / Manually initiate Containment Spray (Alternate Path – Containment Spray Pump Trips)	226.01 M, S, A	5
f. Emergency Generators / Normal start of EDG from control room	264.01 D, S	6
g. Standby Gas Treatment system (SGTS) / Confirm Secondary Containment Initiations and Isolations (Alternate Path – SGTS fails to start) 119' cask ops, RBHVAC trips, SGTS fails to start, verify iso/actuations due to SCC, SP 49.	261.01 N, S, A	9
B.2 Facility Walk-Through	<u> </u>	I
a. Shutdown Cooling / Transfer control to LSP 1A2 (shutdown cooling pump)	308.04 D,R	4 Abnormal
b. Primary Containment / Line-up to vent the Torus through the hardened vent	223.03 D	5 Emergency
c. Instrument Air / Manually scram the reactor by venting the Scram Air Header	279.06 D, R	8 Emergency