WOLF CREEK
NUCLEAR OPERATING CORPORATION

Gary B. Fader
Vice President Technical Services

MAY 21 2002

ET 02-0023

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Subject: Docket No. 50-482: 30-Day Response for NRC Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles"

Gentlemen:

The attachment to this letter contains the Wolf Creek Nuclear Operating Corporation (WCNOC) response to the 30-day requirement of U.S. Nuclear Regulatory Commission (NRC) Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles" dated August 3, 2001. Item 5.a. of the bulletin requires, within 30 days after plant restart following the next refueling outage, information to be provided describing the extent of reactor vessel head penetration nozzle leakage and cracking detected.

No commitments are identified in this submittal. If you should have any questions regarding this submittal, please contact me at (620) 364-4034, or Mr. Tony Harris at (620) 364-4038.

Very truly yours,

Gary B. Fader

GBF/rlr

Attachment

cc: J. N. Donohew (NRC), w/a
D. N. Graves (NRC), w/a
E. W. Merschoff (NRC), w/a
Senior Resident Inspector (NRC), w/a

A088

STATE OF KANSAS)
) **SS**
COUNTY OF COFFEY)

Gary B. Fader, of lawful age, being first duly sworn upon oath says that he is Vice President Technical Services of Wolf Creek Nuclear Operating Corporation; that he has read the foregoing document and knows the contents thereof; that he has executed the same for and on behalf of said Corporation with full power and authority to do so; and that the facts therein stated are true and correct to the best of his knowledge, information and belief.

By _____
Gary B. Fader
Vice President Technical Services

SUBSCRIBED and sworn to before me this 21st day of May, 2002.

_____
Notary Public

NOTARY PUBLIC
STATE OF KANSAS
CINDY NOVINGER
My Appt. Exp. 7/8/02

Expiration Date July 8, 2002

30-Day Response to NRC Bulletin 2001-01
Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles

Below is the Wolf Creek Nuclear Operating Corporation (WCNOC) response to the 30-day requirement of U. S. Nuclear Regulatory Commission (NRC) Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles" dated August 3, 2001. The bulletin's "Requested Information" is shown in bold.

Requested Information

5. Addressees are requested to provide the following information within 30 days after plant restart following the next refueling outage:

a. a description of the extent of VHP nozzle leakage and cracking detected at your plant, including the number, location, size, and nature of each crack detected;

b. if cracking is identified, a description of the inspections (type, scope, qualification requirements, and acceptance criteria), repairs, and other corrective actions you have taken to satisfy applicable regulatory requirements. This information is requested only if there are any changes from prior information submitted in accordance with this bulletin.

Response

Refueling outage 12 ended at Wolf Creek Generating Station (WCGS) on April 27, 2002. During Refuel 12, WCNOC completed a remote visual inspection of the top of the reactor vessel head. This inspection was performed to support an engineering evaluation of the condition of the reactor vessel head relative to the issues addressed in NRC Bulletin 2002-01 (Reference 1). WCGS is considered to have a low susceptibility to primary water stress corrosion cracking (PWSCC), based upon a susceptibility ranking of more than 30 effective full power years from the Oconee Nuclear Station Unit 3 condition. Even though WCGS is considered to have a low susceptibility to PWSCC, the tools, techniques, and procedures employed in this inspection were consistent with the applicable requirements of an "effective visual exam" of the reactor vessel penetration (RVP) nozzles for plants with moderate susceptibility to PWSCC as defined in NRC Bulletin 2001-01 (Reference 2). This inspection was not compromised by the presence of insulation or other restrictions such that the entire circumference of all reactor vessel head penetration nozzles were visually inspected. There were no deposits on the reactor vessel head or any other factors during this inspection that interfered with the detection of potential leakage or other indications on the RVP nozzles. The results of the inspection indicated no evidence of leakage through any RVP nozzle and no evidence of RVP nozzle cracking.

References

1. NRC Bulletin 2002-01, Reactor Pressure Vessel Head Degradation and Reactor Coolant Pressure Boundary Integrity, dated March 18, 2002.

2. NRC Bulletin 2001-01, Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles, dated August 3, 2001.