

November 6, 1991

Docket No. 50-458

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, Louisiana 70775

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Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. 61 TO FACILITY
OPERATING LICENSE NO. NPF-47 (TAC NO. 77577)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 61 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 28, 1990, and supplemented by letters dated February 13, 1991, April 11, 1991, and August 9, 1991.

The amendment revises Technical Specification 6.5.1, "Facility Review Committee," and 6.5.3, "Nuclear Review Board," by deleting the specific composition list for each and replacing it with general statements defining the number of members and the level of expertise for membership. Additional changes include qualification of alternate members, definition of a quorum, and deletion of a reference to the first year of plant operation that has been completed.

A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By

Douglas V. Pickett, Project Manager
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

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Enclosures:

1. Amendment No. 61 to NPF-47
2. Safety Evaluation

cc w/enclosures:

See next page

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Mr. James C. Deddens

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November 6, 1991

cc w/enclosures:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Gulf States Utilities Company (the licensee) dated August 28, 1990, as supplemented by letters dated February 13, 1991, April 11, 1991, and August 9, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 6, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 61

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

REMOVE

6-7
6-8
6-9
6-10
6-11
6-12

INSERT

6-7
6-8
6-9
6-10
6-11
6-12

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the Director-Radiological Programs who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Director - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 FACILITY REVIEW COMMITTEE (FRC)

FUNCTION

6.5.1.1 The FRC shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The FRC shall consist of at least six but not more than eleven members and shall be composed of:

The Chairperson assigned by the Plant Manager. The Chairperson will be chosen from the personnel reporting directly to the Plant Manager.

The remainder of the members shall be from the personnel reporting directly to the Plant Manager and the supervisory staff. This composition will be representative of the various disciplines.

All members shall be qualified to the applicable portions of ANSI/ANS 3.1-1978, Section 4.4, prior to being approved by the Chairperson. This qualification will be maintained while assigned to FRC activities.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the FRC Chairperson and shall be qualified to the applicable portions of ANSI/ANS 3.1-1978, Section 4.4, prior to being approved by the Chairperson. This qualification shall be maintained while assigned to FRC activities.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The FRC shall meet at least once per calendar month and as convened by the FRC Chairperson or designated alternate.

QUORUM

6.5.1.5 The Quorum of the FRC necessary for the performance of the FRC responsibility and authority provisions of these Technical Specifications shall consist of a majority of the members including the Chairperson or Alternate Chairperson, and no more than two alternate members.

RESPONSIBILITIES

6.5.1.6 The FRC shall be responsible for:

- a. Review of all plant general administrative procedures and changes thereto;
- b. Review of all proposed tests and experiments that affect nuclear safety;
- c. Review of all proposed changes to Appendix A Technical Specifications;
- d. Review of all proposed changes or modifications to structures, components, systems or equipment that affect nuclear safety;
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Senior Vice President - RBNG and the Nuclear Review Board;
- f. Review of all REPORTABLE EVENTS;
- g. Review of unit operations to detect potential hazards to nuclear safety; items that may be included in this review are NRC inspection reports that require written response, QA audits/surveillance findings of operating and maintenance activities, NRB audit results, and American Nuclear Insurer (ANI) inspection results;
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant Manager or the Nuclear Review Board;
- i. Review of initial start-up testing phase start-up procedures and revisions; and
- j. Review of the Emergency Plan and implementation procedures at least once per 12 months and all proposed changes thereto.

6.5.1.7 The FRC shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under Specification 6.5.1.6.a. through d. prior to their implementation.
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6.a. through e. constitutes an unreviewed safety question.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- c. Provide written notification within 24 hours to the Senior Vice President - RBNG and the Nuclear Review Board of disagreement between the FRC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.5.1.8 The FRC shall maintain written minutes of each FRC meeting that, at a minimum, document the results of all FRC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Plant Manager and the NRB.

6.5.2 TECHNICAL REVIEW AND CONTROL

6.5.2.1 Each procedure and program required by Specification 6.8 and other procedures that affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group that prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group that prepared the procedure. Each such procedure and program, or changes thereto, shall be approved, prior to implementation, by the Plant Manager, one of the Assistant Plant Managers or the Director - Radiological Programs, or by the manager/department head responsible for the program or the activity described in the procedure, with the exception of the Emergency Plan and implementing procedures which shall be approved by the Manager - Administration, Plant Manager and Senior Vice President - RBNG.

6.5.2.2 Individuals responsible for reviews performed in accordance with Section 6.5.2.1 shall be members of River Bend Nuclear Group supervisory staff, and the reviews shall be performed in accordance with administrative procedures. Each such review shall include a determination of whether or not additional, cross-disciplinary review is necessary and a verification that the proposed actions do not constitute an unreviewed safety question. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

6.5.2.3 The station security program and implementing procedures shall be reviewed at least once per 12 months, and recommended changes approved in accordance with Specification 6.5.2.1.

6.5.2.4 The station emergency plan and implementing procedures and recommended changes shall be approved in accordance with Specification 6.5.2.1.

6.5.2.5 The station fire protection plan and implementing procedures shall be reviewed at least once per 12 months, and recommended changes approved in accordance with Specification 6.5.2.1.

6.5.2.6 Records documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.5 shall be maintained.

ADMINISTRATIVE CONTROLS

6.5.3 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

6.5.3.1 The NRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering,
- h. Quality assurance practices,
- i. Licensing and regulatory affairs,
- j. Training.

The NRB shall report to and advise the Senior Vice President - RBNG on those areas of responsibility in Specifications 6.5.3.7 and 6.5.3.8.

COMPOSITION

6.5.3.2 The NRB shall be composed of at least nine but not more than thirteen individuals who shall possess the necessary expertise to provide the independent review and audit functions identified in Specification 6.5.3.1. All members shall be qualified to the applicable portions of ANSI/ANS 3.1-1978, Section 4.7, prior to being approved by the Chairperson. This qualification will be maintained while assigned to NRB activities. The Senior Vice President-RBNG provides nominations for permanent NRB membership to the NRB Chairperson for review and approval.

ALTERNATES

6.5.3.3 All alternate members shall be appointed in writing by the NRB Chairperson and be qualified to the applicable portions of ANSI/ANS 3.1-1978, Section 4.7, prior to being approved by the Chairperson. This qualification will be maintained while assigned to NRB activities.

CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the NRB Chairperson to provide expert advice to the NRB.

MEETING FREQUENCY

6.5.3.5 The NRB shall meet at least once per 6 months.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.3.6 The Quorum of the NRB necessary for the performance of the review and audit functions of these technical specifications shall consist of a majority of the members including the Chairperson or Vice Chairperson, and no more than two alternate members. No more than a minority of the quorum shall have line responsibility for the operation of the unit.

REVIEW

6.5.3.7 The NRB shall be responsible for the review of:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the FRC.

AUDITS

6.5.3.8 Audits of unit activities shall be performed under the cognizance of the NRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions, at least once per 12 months;
- b. The performance, training and qualifications of the entire unit staff, at least once per 12 months;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection programmatic controls including the implementing procedures, at least once per 24 months by qualified licensee QA personnel;
- f. The fire protection equipment and program implementation, at least once per 12 months, utilizing either qualified offsite licensee personnel or an outside independent fire protection consultant. An outside independent fire protection consultant shall be utilized at least every third year; and
- g. Any other area of unit operation considered appropriate by the NRB, Plant Manager or the Senior Vice President - RBNG.
- h. The Emergency Plan and implementing procedures, at least once per 12 months.
- i. The Security Plan and implementing procedures, at least once per 12 months.
- j. The radiological environmental monitoring program and the results thereof, at least once per 12 months.
- k. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- l. The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes, at least once per 24 months.
- m. The performance of activities by the Quality Assurance Program for effluent and environmental monitoring, at least once per 12 months.

RECORDS

6.5.3.9 Records of NRB activities shall be prepared, approved, and distributed as indicated below:



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 61 TO FACILITY OPERATING LICENSE NO. NPF-47
GULF STATES UTILITIES COMPANY
RIVER BEND STATION, UNIT 1
DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated August 28, 1990, as supplemented by letters dated February 13, 1991, April 11, 1991, and August 9, 1991, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The proposed amendment would delete the specific titles from the composition of the Facility Review Committee (FRC) and the Nuclear Review Board (NRB) and alter the quorum requirements for both of these groups.

During the course of this review the staff objected to a licensee proposal to delete the current limitation on the number of alternates allowed as quorum members on the FRC or NRB. The licensee agreed that the existing limitation of no more than two alternates as members of the FRC or NRB quorum be maintained. Therefore, since this was considered to be a change to the original no significant hazards determination as published in the Federal Register on October 3, 1990 (55 FR 40466), the staff issued a Notice of Withdrawal of Application for Amendment on April 11, 1991 (56 FR 14718). The April 11, 1991, and August 9, 1991, submittals provided additional clarifying information and did not change the initial no significant hazards consideration determination.

2.0 EVALUATION

The following is our evaluation of the amended changes.

- a. Section 6.5.1 - Facility Review Committee (FRC) - Gulf States Utilities proposes to delete the specific titles of the FRC members from the composition and replace the titles with a statement describing the number of members (six but not more than eleven), a reference to the organizational level of committee members, and a statement that all members and alternates, shall be qualified to the applicable portion of Section 4.4 of ANSI/ANS 3.1-1978, prior to being approved by the Chairperson. Section 6.5.1.5, Quorum, has been changed to state that a quorum shall consist of a majority of the members including the Chairperson or alternate Chairperson with no more than two alternate members.

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- b. Section 6.5.3 - Nuclear Review Board (NRB) - Gulf States Utilities proposes to delete the specific titles of the NRB members from the composition and replace the titles with a statement that the NRB shall be composed of at least nine but not more than thirteen members. Section 6.5.3.6, Quorum, has been changed to state that a quorum shall consist of the Chairman or Vice-Chairman and at least a majority of the NRB members with no more than two alternate members. The members and alternates, shall be qualified to the applicable portions of Section 4.7 of ANSI/ANS 3.1-1978.
- c. Section 6.5.3 - Nuclear Review Board - Gulf States Utilities proposes to delete the requirement that the NRB meet at least once per calendar quarter during the initial year of unit operation following fuel load.

We find the requested changes for items 1 and 2 above acceptable as they meet the appropriate acceptance criteria of Section 13.4 of the Standard Review Plan (NUREG-0800). With regard to item c., we find this change acceptable because the unit has completed the initial year of operation thus making this specification moot.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Frederick R. Allenspach, LPEB/NRR

Date: November 6, 1991