

December 13, 1988

Docket No. 50-458

Gulf States Utilities  
ATTN: Mr. James C. Deddens  
Senior Vice President (RBNG)  
Post Office Box 220  
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. 32 TO FACILITY  
OPERATING LICENSE NO. NPF-47 (TAC NO. 69096)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 32 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 12, 1988.

The amendment reduces the minimum water coverage above a spent fuel assembly in the storage pool and above an irradiated control rod during spent fuel and irradiated control rod handling operations. The reduced minimum water coverage for irradiated control rods also permits placing these rods on hangers provided for their storage in the fuel building pool.

A copy of our Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

/s/

Walter A. Paulson, Project Manager  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

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Enclosures:

1. Amendment No. 32 to License No. NPF-47
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:  
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DOCUMENT NAME: RB AMENDMENT TAC 69096

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| PNoonan  | WPaulson: |               |
| 11/22/88 | 11/23/88  | 11/24/88      |

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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The amendment reduces the minimum water coverage above a spent fuel assembly in the storage pool and above an irradiated control rod during spent fuel and irradiated control rod handling operations. The reduced minimum water coverage for irradiated control rods also permits placing these rods on hangers provided for their storage in the fuel building pool.

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*Walter A. Paulson*

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Division of Reactor Projects - III,  
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Office of Nuclear Reactor Regulation

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See next page

Mr. James C. Deddens  
Gulf States Utilities Company

River Bend Nuclear Plant

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 32  
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Gulf States Utilities Company (the licensee) dated August 12, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 32 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 13, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 32

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contains a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.

REMOVE PAGES

3/4 9-8  
3/4 9-9

INSERT PAGES

3/4 9-8  
3/4 9-9

## REFUELING OPERATIONS

### 3/4.9.5 COMMUNICATIONS

#### LIMITING CONDITION FOR OPERATION

---

3.9.5 Direct communication shall be maintained between the main control room and refueling platform personnel.

APPLICABILITY: OPERATIONAL CONDITION 5, during CORE ALTERATIONS.

#### ACTION:

When direct communication between the main control room and refueling platform personnel cannot be maintained, immediately suspend CORE ALTERATIONS.

#### SURVEILLANCE REQUIREMENTS

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4.9.5 Direct communication between the main control room and refueling platform personnel shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.

## REFUELING OPERATIONS

### 3/4.9.6 REFUELING AND FUEL HANDLING PLATFORMS

#### REFUELING PLATFORM

##### LIMITING CONDITION FOR OPERATION

3.9.6.1 The refueling platform shall be OPERABLE and used for handling fuel assemblies or control rods.

APPLICABILITY: During handling of fuel assemblies or control rods.

##### ACTION:

With the requirements for refueling platform OPERABILITY not satisfied, suspend use of any inoperable refueling platform equipment from operations involving the handling of control rods and fuel assemblies after placing the load in a safe condition.

##### SURVEILLANCE REQUIREMENTS

4.9.6.1 Each refueling platform hoist used for handling of control rods or fuel assemblies shall be demonstrated OPERABLE within 7 days prior to the start of such operations with that hoist by:

- a. Demonstrating operation of the overload cutoff on the main hoist before the load exceeds  $1200 \pm 50$  pounds.
- b. Demonstrating operation of the overload cutoff on the frame-mounted and monorail-mounted auxiliary hoists when the load exceeds  $500 \pm 50$  pounds.
- c. Demonstrate operation of the normal uptravel stop interlock of the main hoist to maintain at least 8 feet 2 inches of water coverage above the top of the active irradiated fuel.
- d. Demonstrate operation of the normal uptravel stop interlock of the frame-mounted and monorail-mounted auxiliary hoists to maintain at least 6 feet 9 inches of water coverage above the top of the irradiated control rod.
- e. Demonstrating operation of the downtravel interlock on the main hoist when grapple hook down travel reaches 4 inches below fuel assembly handle during operations within the reactor pressure vessel.
- f. Demonstrating operation of the slack cable cutoff on the main hoist when the load is less than  $50 \pm 10$  pounds.
- g. Demonstrating operation of the loaded interlock on the main hoist when the load exceeds  $485 \pm 50$  pounds.
- h. Demonstrating operation of the redundant loaded interlock on the main hoist when the load exceeds  $550 \pm 50$  pounds.

## REFUELING OPERATIONS

### 3/4.9.6 REFUELING AND FUEL HANDLING PLATFORMS

#### FUEL HANDLING PLATFORM

#### LIMITING CONDITION FOR OPERATION

---

3.9.6.2 The fuel handling platform shall be OPERABLE and used for handling fuel assemblies or control rods.

APPLICABILITY: During handling of fuel assemblies or control rods.

#### ACTION:

With the requirements for fuel handling platform OPERABILITY not satisfied, suspend use of any inoperable refueling platform equipment from operations involving the handling of control rods and fuel assemblies after placing the load in a safe condition.

#### SURVEILLANCE REQUIREMENTS

---

4.9.6.2 Each fuel handling platform hoist used for handling of control rods or fuel assemblies shall be demonstrated OPERABLE within 7 days prior to the start of such operations with that hoist by:

- a. Demonstrating operation of the overload cutoff on the main hoist before the load exceeds  $1100 \pm 50$  pounds.
- b. Demonstrating operation of the overload cutoff on the monorail-mounted auxiliary hoists when handling control rods before the load exceeds  $500 \pm 50$  pounds, and when handling unirradiated fuel before the load exceeds  $1000 \pm 50$  pounds.
- c. Demonstrating operation of the normal uptravel stop interlock of the main hoist to maintain at least 8 feet 2 inches of water coverage above the top of the active irradiated fuel.
- d. Demonstrate operation of the normal uptravel stop interlock of the monorail-mounted auxiliary hoist to maintain at least 6 feet 9 inches of water coverage above the top of the irradiated control rod.
- e. Demonstrating operation of the slack cable cutoff on the main hoist when the load is less than  $50 \pm 10$  pounds.
- f. Demonstrating operation of the loaded interlock on the main hoist when the load exceeds  $350 \pm 50$  pounds.

## REFUELING OPERATIONS

### 3/4.9.7 CRANE TRAVEL - SPENT AND NEW FUEL STORAGE, TRANSFER AND UPPER CONTAINMENT FUEL POOLS

#### LIMITING CONDITION FOR OPERATION

---

3.9.7 Loads in excess of 1200 pounds shall be prohibited from travel over fuel assemblies in the spent or new fuel storage, transfer or upper containment fuel pool racks.

**APPLICABILITY:** With fuel assemblies in the spent or new fuel storage, transfer or upper containment fuel pools.

#### ACTION:

With the requirements of the above specification not satisfied, place the crane load in a safe condition. The provisions of Specification 3.0.3 are not applicable.

## SURVEILLANCE REQUIREMENTS

---

4.9.7.1 The fuel building crane loads shall be verified to weigh less than or equal to 1200 pounds before travel over fuel assemblies in the spent or new fuel storage pools and the lower transfer pools.

4.9.7.2 The reactor building polar crane loads shall be verified to weigh less than or equal to 1200 pounds before travel over fuel assemblies in the upper transfer and containment fuel pools.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO.32 TO FACILITY OPERATING LICENSE NO. NPF-47

GULF STATES UTILITIES COMPANY

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated August 12, 1988, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The proposed amendment would reduce the minimum water coverage over the top of active irradiated fuel from 8 feet 6 inches to 8 feet 2 inches, and reduce the minimum water coverage over irradiated control rods from 8 feet 6 inches to 6 feet 9 inches. The surveillance requirements of Technical Specifications (TS) 4.9.6.1 and 4.9.6.2 would be revised to be consistent with these revised minimum water coverage values with regard to demonstrating the operation of the normal uptravel stop interlocks of the appropriate refueling platform and fuel handling platform hoists.

2.0 DISCUSSION AND EVALUATION

The proposed reduction in minimum water coverage would relieve the spent fuel operator from unnecessary hardship due to the narrow operating bands for pool levels in the containment and fuel building. The narrow operating bands have made it difficult to maintain the pool levels where the high or low water level annunciators are not actuated.

The proposed minimum water coverage over the spent fuel and irradiated control rods would permit an easier transfer through the fuel transfer shield, and would permit placing control rods on hangers provided for their storage in the fuel building pool. In order to accomplish this, the proposed TS change necessitates a separation of the water coverage into two requirements; one for irradiated fuel, and the other for irradiated control rods. The first surveillance requirement would be changed to permit the main hoist normal uptravel stop limit switch to provide at least 8 feet 2 inches of water coverage over the top of active irradiated fuel, and the second surveillance requirement would permit that the auxiliary hoist normal uptravel stop limit switch to provide at least 6 feet 9 inches of water coverage over the top of irradiated control rod.

The licensee estimated a radiation dose rate increase to personnel on the refueling platform of 0.2 mrem per hour due to the reduction of water coverage above the active zone of the irradiated fuel (decayed 24 hours), and 0.3 mrem per hour from an irradiated control rod, assuming pool water

level at mid-range. The licensee also estimated that personnel radiation exposure over the plant design life would increase by 0.2 man-Rem from irradiated fuel, and 0.01 man-Rem from irradiated control rods. The relatively low personnel exposure during the plant life from irradiated control rods is because of the "control cell" core concept currently used at River Bend Station. During the first nine or ten refueling outages, control rods will be shuffled within the core to equalize their exposure. At the tenth or eleventh outage, 29 control rods will be removed and replaced. For this reason, because of the reduced frequency and duration of control rod handling, the proposed change will have only a minimal impact on personnel radiation exposure.

The pool water level (as well as the water level above the RPV flange) will remain unchanged, with a minimum of 23 feet above the spent fuel in the storage racks (the volume of the pool water will remain unchanged for the purpose of fuel handling accident calculation); therefore, it can be concluded that the proposed change would not increase the probability of the consequences of any accident previously evaluated.

The proposed change would not create the possibility of a new, or a different kind of accident, than any previously evaluated, since the proposed change is limited to water coverage above spent fuel and irradiated control rods during handling in the containment and fuel handling pools. It would not involve a significant reduction in the margin of safety, and also the offsite dose consequences of a postulated fuel handling accident would remain well within the 10 CFR 100 guidelines, and would not be affected by the proposed Technical Specification change.

On the basis of the above evaluation, the staff concludes that the TS changes proposed by Gulf States Utilities for the River Bend Station, Unit 1, are consistent with 10 CFR Part 20, 10 CFR 51.31, and Regulatory Guide 8.8, and therefore are acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR Parts 51.21, 51.32 and 51.35, an environmental assessment and finding of no significant impact was published in the Federal Register on December 13, 1988 (53 FR 50140).

Accordingly, based upon the environmental assessment, the Commission has determined that issuance of this amendment will not have a significant effect on the quality of the human environment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations,

and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. The staff therefore concludes that the proposed changes are acceptable, and they are hereby incorporated into the River Bend Unit 1 Technical Specifications.

Dated: December 13, 1988

Principal Contributor: F. Skopec

UNITED STATES NUCLEAR REGULATORY COMMISSIONGULF STATES UTILITIES COMPANYDOCKET NO. 50-458NOTICE OF ISSUANCE OF AMENDMENTTO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 32 to Facility Operating License No. NPF-47, issued to Gulf States Utilities Company, (the licensee), which revised the Technical Specifications for operation of the River Bend Station, Unit 1. located in West Feliciana Parish, Louisiana.

The amendment was effective as of the date of its issuance.

The amendment revised the Technical Specifications to reduce the minimum water coverage above a spent fuel assembly in the storage pool and above an irradiated control rod during spent fuel and irradiated control rod handling. The reduced minimum water coverage for irradiated control rods also permits placing these rods on hangers provided for their storage in the fuel building pool.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

The Notice of Consideration of Issuance of Amendment was published in the FEDERAL REGISTER on August 25, 1988 (53 FR 32484) and a correction was

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published in the FEDERAL REGISTER on October 24, 1988 (53 FR 41634). No request for a hearing or petition for leave to intervene was filed following the notices.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment.

For further details with respect to the action, see: (1) the application for amendment dated August 12, 1988; (2) Amendment No. 32 to Facility Operating License No. NPF-47; and (3) the Commission's related Safety Evaluation and Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, 2120 L Street, N.W., Washington, D.C. 20555; at Government Documents Department, Louisiana State University, Baton Rouge, Louisiana 70803. A copy of items (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects - III, IV, V and Special Projects.

Dated at Rockville, Maryland this 13th day of December, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

*Walter A. Paulson*

Walter A. Paulson, Project Manager  
Project Director - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation