

March 3, 1989

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. 36 TO FACILITY
OPERATING LICENSE NO. NPF-47 (TAC NO. 71749)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 36 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 16, 1988 as modified January 24, 1989.

This amendment modifies the TSs to add one-time exceptions to TS 3.0.4 for use during the second refueling outage. The exceptions will allow entry into specified operational conditions without meeting the Limiting Condition for Operation, provided the requirements of associated action statements are met.

A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 36 to License No. NPF-47
2. Safety Evaluation

cc w/enclosures:
See next page

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DOCUMENT NAME: RIVER BEND TAC NO 71749

PD4/LA <i>[initials]</i>	PD4/PM <i>[initials]</i>	PLB/C <i>[initials]</i>	SRB/C <i>[initials]</i>	OGC-Rockville	PD4/D
PNoonan	WPaulson:	UCraig	MHodges		JCalvo
2/8/89	2/17/89	2/10/89	2/13/89	2/17/89	2/3/89

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

March 3, 1989

Docket No. 50-458

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, LA 70775

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A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Walter A. Paulson".

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 36 to
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2. Safety Evaluation

cc w/enclosures:
See next page

Mr. James C. Deddens
Gulf States Utilities Company

River Bend Nuclear Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Gulf States Utilities Company dated December 16, 1988, as modified January 24, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

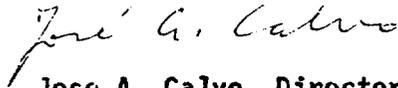
(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 36 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 3, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.

REMOVE PAGES

3/4 4-29
3/4 7-5
3/4 9-18

INSERT PAGES

3/4 4-29
3/4 7-5
3/4 9-18

REACTOR COOLANT SYSTEM

COLD SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.9.2 Two# shutdown cooling mode loops of the residual heat removal (RHR) system shall be OPERABLE and, unless at least one recirculation pump is in operation, at least one shutdown cooling mode loop shall be in operation*,## with each loop consisting of at least:

- a. One OPERABLE RHR pump, and
- b. Two OPERABLE RHR heat exchangers.

APPLICABILITY: OPERATIONAL CONDITION 4.

ACTION:

- a. With less than the above required RHR shutdown cooling mode loops OPERABLE, within one hour and at least once per 24 hours thereafter demonstrate the operability of at least one alternate method capable of decay heat removal for each inoperable RHR shutdown cooling mode loop.
- b. With no RHR shutdown cooling mode loop or recirculation pump in operation, within one hour establish reactor coolant circulation by an alternate method and monitor reactor coolant temperature and pressure at least once per hour.
- c. The provisions of Specification 3.0.4 are not applicable.**

SURVEILLANCE REQUIREMENTS

4.4.9.2 At least once per 12 hours, at least one shutdown cooling mode loop of the residual heat removal system, one recirculation pump or alternate method shall be determined to be in operation and circulating reactor coolant.

#One RHR shutdown cooling mode loop may be inoperable for up to 2 hours for surveillance testing provided the other loop is OPERABLE and in operation or at least one recirculation pump is in operation.

*The shutdown cooling pump may be removed from operation for up to 2 hours per 8 hour period provided the other loop is OPERABLE.

##The shutdown cooling mode loop may be removed from operation during hydrostatic testing.

**This exception is applicable until startup from the second refueling outage.

PLANT SYSTEMS

3/4.7.2 MAIN CONTROL ROOM AIR CONDITIONING SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.2 The main control room air conditioning system, with two independent air handling unit/filter train subsystems, shall be OPERABLE.

APPLICABILITY: All OPERATIONAL CONDITIONS and *.

ACTION:

- a. In OPERATIONAL CONDITION 1, 2 or 3 with one main control room air conditioning subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION 4, 5 or *:
 1. With one main control room air conditioning air handling/filter train subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or initiate and maintain operation of the OPERABLE subsystem in the emergency mode of operation.
 2. With both main control room air conditioning air handling/filter train subsystems inoperable, suspend CORE ALTERATIONS, handling of irradiated fuel in the primary containment and Fuel Building and operations with a potential for draining the reactor vessel.
 3. The provisions of Specification 3.0.4 are not applicable.#
- c. The provisions of Specification 3.0.3 are not applicable in Operational Condition *.

SURVEILLANCE REQUIREMENTS

4.7.2 Each main control room air conditioning subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 104°F.
- b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the subsystem operates for at least 10 hours with the heaters OPERABLE.

*When irradiated fuel is being handled in the primary containment or Fuel Building.

#This exception is applicable until startup from the second refueling outage.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the subsystem by:
1. Verifying that the subsystem satisfies the in-place penetration and bypass leakage testing acceptance criterion of less than 0.05% and uses the test procedure guidance in Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the system flow rate is 4000 cfm \pm 10%.
 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978, for a methyl iodide penetration of less than 0.175%; and
 3. Verifying a subsystem flow rate of 4000 cfm \pm 10% during subsystem operation when tested in accordance with ANSI N510-1980.
- d. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978, for a methyl iodide penetration of less than 0.175%.
- e. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 8 inches water gauge while operating the subsystem at a flow rate of 4000 cfm \pm 10%.
 2. Verifying that on each of the below emergency mode actuation test signals, the subsystem automatically switches to the emergency mode of operation, the isolation valves close within 30 seconds, and the control room is maintained at a positive pressure of $> 1/8$ inch water gauge relative to the outside atmosphere during subsystem operation at a flow rate less than or equal to 4,000 cfm:
 - **a) LOCA, and
 - b) Local air intake radiation monitor - High.
 3. Verifying that the heaters dissipate 23 ± 2.3 kw when tested in accordance with ANSI N510-1980, at the design supply voltage.

**The specified 18 month interval during the first operating cycle may be extended to coincide with completion of the first refueling outage, scheduled to begin 9-15-87.

REFUELING OPERATIONS

LOW WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.11.2 Two shutdown cooling mode loops of the residual heat removal (RHR) system shall be OPERABLE and at least one loop shall be in operation,* with each train consisting of at least:

- a. One OPERABLE RHR pump, and
- b. Two OPERABLE RHR heat exchangers.

APPLICABILITY: OPERATIONAL CONDITION 5, when irradiated fuel is in the reactor vessel and the water level is less than 23 feet above the top of the reactor pressure vessel flange.

ACTION:

- a. With less than the above required shutdown cooling mode loops of the RHR system OPERABLE, within one hour and at least once per 24 hours thereafter, demonstrate the operability of at least one alternate method capable of decay heat removal for each inoperable RHR shutdown cooling mode train.
- b. With no RHR shutdown cooling mode loop in operation, within one hour establish reactor coolant circulation by an alternate method, and monitor reactor coolant temperature at least once per hour.
- c. The provisions of Specification 3.0.4 are not applicable.**

SURVEILLANCE REQUIREMENTS

4.9.11.2 At least one shutdown cooling mode loop of the residual heat removal system or alternate method shall be verified, at least once per 12 hours, to be in operation and circulating reactor coolant.

*The shutdown cooling pump may be removed from operation for up to 2 hours per 8-hour period.

**This exception is applicable until startup from the second refueling outage.

REFUELING OPERATIONS

3/4.9.11 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION

HIGH WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.11.1 At least one shutdown cooling mode loop of the residual heat removal (RHR) system shall be OPERABLE and in operation* with at least:

- a. One OPERABLE RHR pump, and
- b. Two OPERABLE RHR heat exchangers.

APPLICABILITY: OPERATIONAL CONDITION 5, when irradiated fuel is in the reactor vessel and the water level is greater than or equal to 23 feet above the top of the reactor pressure vessel flange.

ACTION:

- a. With no RHR shutdown cooling mode OPERABLE, within one hour and at least once per 24 hours thereafter, demonstrate the operability of at least one alternate method capable of decay heat removal. Otherwise, suspend all operations involving an increase in the reactor decay heat load and establish PRIMARY CONTAINMENT INTEGRITY - FUEL HANDLING within 4 hours.
- b. With no RHR shutdown cooling mode loop in operation, within one hour establish reactor coolant circulation by an alternate method, and monitor reactor coolant temperature at least once per hour.

SURVEILLANCE REQUIREMENTS

4.9.11.1 At least one shutdown cooling mode loop of the residual heat removal system or alternate method shall be verified, at least once per 12 hours, to be in operation and circulating reactor coolant.

*The shutdown cooling loop may be removed from operation for up to 2 hours per 8-hour period.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. NPF-47

GULF STATES UTILITIES COMPANY

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated December 16, 1988, as modified January 24, 1989, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The proposed amendment would provide one-time exceptions to Section 3.0.4 of the Technical Specifications (TSs) for use during the second refueling outage. The one-time exceptions would be added to TSs 3.4.9.2, Reactor Coolant System, Cold Shutdown; 3.7.2, Plant Systems, Main Control Room Air Conditioning System, and 3.9.11.2, Refueling Operations, Low Water Level. The exceptions would allow entry into certain operational conditions without meeting the Limiting Condition for Operation (LCO), provided the requirements of the associated action statements are met.

2.0 EVALUATION

Section 3.0.4 of the TSs states:

3.0.4 Entry into an OPERATIONAL CONDITION or other specified condition shall not be made unless the conditions for the Limiting Condition for Operation are met without reliance on provisions contained in the ACTION requirements. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements. Exceptions to these requirements are stated in the individual Specifications.

The proposed changes to the TSs would provide one-time exceptions to TS 3.0.4 during the second refueling outage for the following three TSs:

1. TS 3.4.9.2. Action "c" would be added which states that the provisions of Specification 3.0.4 are not applicable. A footnote would also be added that states that this exception is applicable until startup from the second refueling outage.
2. TS 3.7.2 Action "b" would be added which states that the provisions of Specification 3.0.4 are not applicable. This action statement applies in Operational Condition 4 or 5 or when irradiated fuel is being handled in the primary Containment or Fuel Building. A footnote would also be added that states that this exception is applicable until startup from the second refueling outage.

3. TS 3.9.11.2 Action "c" would be added which states that the provisions of Specification 3.0.4 are not applicable. A footnote would also be added that states that this exception is applicable until startup from the second refueling outage.

During the second refueling outage, presently scheduled to begin March 15, 1989, residual heat removal (RHR) trains and the control building chilled water system trains will be made inoperable at various times to perform required maintenance, surveillance testing and to make modifications to the main control room air conditioning system. These activities will require the plant to enter action statements for shutdown cooling and main control room air conditioning at various times during the outage. The proposed changes will provide one-time exceptions to TS 3.0.4 for these specifications for the second refueling outage only, to allow the plant to enter Operational Conditions 4 and 5 to allow reactor head detensioning and tensioning, reactor cavity draining, and to upgrade the capacity of the control building chilled water system. With the present TSs, these RHR maintenance and testing activities and the upgrading of the control building chilled water system would have to be interrupted prior to changing operational conditions to make the RHR and main control room air conditioning systems operable in accordance with the LCOs of the TSs. After entering Operational Condition 4 or 5, the RHR and main control room air conditioning systems would again be made inoperable and the action statements entered to complete the maintenance, testing, and modification activities.

In addition to the TS requirements, the licensee's outage philosophy at River Bend Station (RBS) is to schedule systems out of service for maintenance, modification and testing on a divisional basis. As a result of this, at least one division of all required equipment is scheduled to be operable as required by Technical Specification LCOs and ACTION statements unless required maintenance or testing precludes this. Because of the divisional outage philosophy at RBS, the diesel generators will normally be scheduled to be removed from service for maintenance and surveillance testing when the respective division of equipment is scheduled to be removed from service. Additionally, both loops of the safety-related spent fuel pool cooling system will normally be operable unless required maintenance or testing precludes this.

The licensee's submittals describe the use of proposed exceptions in the second refueling outage and the alternate methods of decay heat removal and operation of the main control room air conditioning system to meet the action statement requirements. The dates are based on the current schedule for the second refueling outage.

1. Shutdown Cooling Mode Loops of the Residual Heat Removal (RHR) System: TSs 3.4.9.2 and 3.9.11.2.

After entering Operational Condition 4 on March 15, 1989, the "A" feedwater line will be removed from service for maintenance. This action also removes the normal return for the "A" RHR shutdown cooling mode loop from service. While the "A" RHR shutdown cooling mode loop will otherwise be operable, the "A" RHR shutdown cooling

mode loop will have a scheduled alternate return path through the LPCI injection nozzle. Thus, the "A" RHR shutdown cooling mode loop with its alternate return path will be considered as an alternate method of decay heat removal in accordance with the Technical Specification ACTION requirements. This configuration is scheduled to remain in effect until March 18, 1989 after the reactor is in Operational Condition 5 with the reactor cavity flooded. Upon reaching this condition, only one RHR shutdown cooling mode loop is required to be operable per Technical Specification 3.9.11.1 and the "A" RHR shutdown cooling mode loop with its alternate return path will then be made inoperable to perform leak rate testing. Following leak rate testing and system maintenance, the "A" RHR shutdown cooling mode loop will be restored to operable status on April 5, 1989.

Both RHR shutdown cooling mode loops are scheduled to be out of service between April 17, 1989 and April 20, 1989 in order to perform leak rate testing on the RHR shutdown cooling suction line isolation valves. This leak rate testing will be performed prior to draining the reactor cavity. Following these leak rate tests, the "A" RHR shutdown cooling mode loop will be restored to operable status and placed in standby. Both RHR shutdown cooling mode loops are again scheduled to be removed from service between April 24, 1989 and April 25, 1989 to perform the reactor vessel hydrostatic test. Following reactor vessel hydrostatic testing, the "A" RHR shutdown cooling mode loop will be returned to operable status on April 25, 1989 and the "B" RHR shutdown cooling mode loop will be returned to service on April 26, 1989. During the time beginning with the leak rate testing on the RHR shutdown cooling suction line isolation valves on April 17, 1989 through the completion of the reactor vessel hydrostatic testing and restoring both RHR shutdown cooling mode loops to operable status of April 26, 1989, the reactor water cleanup (RWCU) system is scheduled to be in service to provide the required decay heat removal capability. The RWCU system can only be taken credit for being an alternate method of decay heat removal for meeting these ACTION requirements when this system has enough decay heat removal capability for the plant conditions at that time. As discussed in the December 16, 1988 submittal, the licensee has calculated the decay heat removal requirements versus time after shutdown and incorporated these results in Surveillance Test Procedure, STP-204-0700, "Alternate Decay Heat Removal Verification." The required decay heat removal is obtained from this procedure and the alternate method of decay heat removal is selected from a table that provides the decay heat removal capability of the alternate method. The adequacy of heat removal of the alternate method is then verified after it is put into service. Alternate methods for coolant circulation include operation of one or both recirculation pumps while in Operational Condition 5, operation of the RWCU in conjunction with maximum control rod drive water flow, or establishing natural core circulation by increasing reactor water level.

The "B" RHR shutdown cooling mode loop is scheduled to remain in operation from the start of the refueling outage until the "A" RHR shutdown cooling mode loop is restored to operable status on April 5, 1989. At that time, the "B" RHR shutdown cooling mode loop will be removed from service to perform leak rate testing and system maintenance. As stated above, the "B" RHR shutdown cooling mode loop will remain out of service through the leak rate testing of the RHR shutdown cooling suction line isolation valves beginning on April 17, 1989 and completion of the reactor vessel hydrostatic testing. Following completion of the reactor vessel hydrostatic testing, the "B" RHR shutdown cooling mode loop will be returned to service on April 26, 1989.

The proposed exceptions to Specification 3.0.4 for RHR shutdown cooling will be utilized:

- 1) on March 18, 1989 when the "A" RHR shutdown cooling mode loop is scheduled to be inoperable, the reactor vessel head is scheduled to be removed and a change to Operational Condition 5 with reactor cavity level less than 23 feet above the top of the reactor pressure vessel flange is scheduled. Technical Specification 3.9.11.2 requires both RHR shutdown cooling mode loops to be operable. At that time it is scheduled to have the "B" RHR shutdown cooling mode loop operable and in operation and the "A" RHR shutdown cooling mode loop with an alternate return path through the LPCI injection nozzle providing the alternate method of decay heat removal required by the ACTION statements. Therefore, the proposed exception to Specification 3.0.4 is required to change specified conditions as currently scheduled.
- 2) on April 21, 1989 when the "B" RHR shutdown cooling mode loop is scheduled to be inoperable, the reactor cavity is scheduled to be drained and a change to Operational Condition 5 with reactor cavity level less than 23 feet above the top of the reactor pressure vessel flange is scheduled. Technical Specification 3.9.11.2 requires both RHR shutdown cooling mode loops to be operable. At that time it is scheduled to have the "A" RHR shutdown cooling mode loop operable with the RWCU system in service providing the alternate method of decay heat removal required by the ACTION statements. Therefore, the proposed exception to Specification 3.0.4 is required to change specified conditions as currently scheduled.
- 3) on April 24, 1989 when the "B" RHR shutdown cooling mode loop is scheduled to be inoperable, the reactor vessel head is scheduled to be reinstalled and a change to Operational Condition 4 is scheduled. Technical Specification 3.4.9.2 also requires both RHR shutdown cooling mode loops to be operable. Again it is scheduled to have the "A" RHR shutdown cooling mode loop operable with the RWCU system in service providing the alternate method of decay heat removal required by the ACTION

statements. Therefore, the proposed exception to Specification 3.0.4 is required to change Operational Conditions as currently scheduled.

Based on the above schedule, at least one RHR shutdown cooling mode loop is scheduled to be available at all times except between April 17, 1989 and April 21, 1989 during the leak rate testing of the RHR shutdown cooling suction valves with the reactor cavity flooded and between April 24, 1989 and April 25, 1989 during the reactor pressure vessel hydrostatic testing with the reactor in Operational Condition 4. During these times, the RWCU system is scheduled to be the only decay heat removal system in service. While this condition is allowed by the current Technical Specifications, the effect of the loss of the RWCU system has been considered. During this time the "A" RHR shutdown cooling mode is scheduled to be available if needed. This system will not be in service during these times because the required leak rate and reactor pressure vessel hydrostatic testing precludes this. However, in the event of the loss of the RWCU system, the "A" RHR shutdown cooling mode loop can be placed into service after terminating the leak rate testing or depressurizing the reactor vessel.

The effects of a loss of offsite power have also been considered. During the time that the Division I diesel generator will be inoperable (between March 15, 1989 and April 3, 1989) the Division II diesel generator is scheduled to be operable and in operation. Therefore, there will be no effect on decay heat removal capability in the event of a loss of offsite power during this time period. During the time that the Division II diesel generator will be inoperable (between April 3, 1989 and April 23, 1989) the "A" RHR shutdown cooling mode loop is scheduled to be operable (between April 5, 1989 and April 17, 1989, between April 21, 1989 and April 24, 1989 and after April 25, 1989) or available as described above (between April 17, 1989 and April 21, 1989 and between April 24, 1989 and April 25, 1989) except for the time between April 3, 1989 and April 5, 1989. During the time the Division II diesel generator is inoperable, the Division I diesel generator is scheduled to be operable. Therefore, except for the time between April 3, 1989 and April 5, 1989, the "A" RHR shutdown cooling mode loop can be made available and there will be no effect on decay heat removal capability in the event of a loss of offsite power.

During the time between April 3, 1989 and April 5, 1989, the "B" RHR shutdown cooling mode loop is scheduled to be in operation to provide the required decay heat removal capability and the Division II diesel generator is scheduled to be inoperable. While this configuration is allowed by the current Technical Specifications, the effect of a loss of offsite power while in this configuration was also considered. During this time, both divisions of the safety-related spent fuel pool cooling system are scheduled to be operable, the reactor will be in Operational Condition 5 with

the reactor cavity flooded and fuel movement is scheduled to be completed. Therefore, decay heat loads from the reactor core will be much reduced since approximately one-third of the irradiated fuel will have been replaced with new fuel. In the event of a loss of offsite power, the safety-related spent fuel pool cooling system can be restored to service for cooling the spent fuel pool. The decay heat in the reactor vessel can be removed using a feed and bleed type decay heat removal method, injecting water into the reactor cavity or vessel and draining excess water to the suppression pool. In the event of a loss of offsite power, ECCS pumps, ECCS jockey pumps, standby service water pumps and/or diesel driven fire pumps can be used. Drain paths can be established through normal cavity drains, the RWCU system, the RHR system or other means.

2. Main Control Room Air Conditioning System: Technical Specification 3.7.2.

During the second refueling outage, a modification will be installed to upgrade the capacity of the control building chilled water system to two 100 percent chillers and two 100 percent chilled water pumps per division. To perform this modification, both divisions of the control building chilled water system must separately be made inoperable. This will cause the respective division of the main control room air conditioning system to become inoperable. Technical Specification 3.7.2 requires both subsystems to be operable in all Operational Conditions and when irradiated fuel is being handled in the primary containment or fueling building. The ACTION statements allow one subsystem to be inoperable for up to seven days. If this allowable out-of-service time is exceeded, it is required to initiate and maintain the operable subsystem in the emergency mode of operation. If both subsystems are inoperable, it is required to suspend CORE ALTERATIONS, handling of irradiated fuel in the primary containment or fuel building and operations with a potential for draining the reactor vessel.

The licensee stated that much of the installation work is scheduled to be performed prior to the refueling outage and every effort will be made to minimize the installation time of the modification. The outage required modifications for Division I are scheduled to be implemented between March 15, 1989 and March 27, 1989 and for Division II between April 4, 1989 and April 16, 1989. These system windows include required system maintenance and testing as well as installation of the above modification. When the division has been inoperable for seven days, the operable main control room air conditioning system will be placed into operation in the emergency mode in accordance with the ACTION requirements. The licensee stated that at no time are both divisions of the main control room air conditioning system scheduled to be inoperable at the same time. However, due to the large scope and complex nature of this modification, there is a high probability that the installation schedule may slip past the scheduled date of April 24, 1989 for entry into Operational Condition 4. The licensee indicated that this proposed exception is being requested

as a reasonable contingency to avoid the likely event that an emergency Technical Specification change would be required to be submitted should the scheduled activities for Division II slip past April 24, 1989.

The NRC staff has reviewed the licensee's submittals requesting one-time exceptions to TS 3.0.4 during the second refueling outage. The licensee has planned the outage to minimize the time when the RHR trains and the main control room air conditioning system will be inoperable because of required maintenance, testing, and system modifications. The licensee has analyzed the decay heat requirements as a function of time after shutdown. The decay heat removal capability of approved alternate methods is provided in a surveillance test procedure. Verification of adequate decay heat removal is required after the alternate method is placed in operation.

The use of the proposed exceptions to TS 3.0.4 in the manner proposed by the licensee will not change the safety margins for shutdown cooling and main control room air conditioning which would be available without the use of these exceptions. Accordingly, the staff concludes that the one-time exceptions to TS 3.0.4 proposed by the licensee for use during the second refueling outage at River Bend Station, Unit 1, are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. The staff therefore concludes that the proposed changes are acceptable, and they are hereby incorporated into the River Bend Unit 1 Technical Specifications.

Dated: March 3, 1989

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