

May 10, 1988

Docket No. 50-458

Gulf States Utilities  
ATTN: Mr. James C. Deddens  
Senior Vice President (RBNG)  
Post Office Box 220  
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. 22 TO FACILITY  
OPERATING LICENSE NO. NPF-47 (TAC NO. 67495)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 22 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 18, 1988 as revised March 22, 1988.

The amendment modifies TS 3.4.2.1 to change the setpoint tolerance of the safety valve function of the safety relief valves from  $\pm 1\%$  to  $(+)0(-)2\%$  of the set pressure.

A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Walter A. Paulson, Project Manager  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 22 to License No. NPF-47
2. Safety Evaluation

cc w/enclosures:  
See next page

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Sincerely,

/s/

Walter A. Paulson, Project Manager  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

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*W.A. Paulson*  
3/15/88

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5/10/88



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
May 10, 1988

Docket No. 50-458

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Senior Vice President (RENG)  
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A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Walter A. Paulson".

Walter A. Paulson, Project Manager  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

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2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. James C. Deddens  
Gulf States Utilities Company

River Bend Nuclear Plant

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22  
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Gulf States Utilities Company (the licensee) dated February 18, 1988 as revised March 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 22 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Jose A. Calvo*

Jose A. Calvo, Director  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Charges to the Technical  
Specifications

Date of Issuance: May 10, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 22

FACILITY/OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.

REMOVE PAGES

3/4 4-7

INSERT PAGES

3/4 4-7

## REACTOR COOLANT SYSTEM

### 3/4.4.2 SAFETY VALVES

#### SAFETY/RELIEF VALVES

#### LIMITING CONDITION FOR OPERATION

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3.4.2.1 The safety valve function of at least 5 of the following valves and the relief valve function of at least 4 additional valves of the following valves, other than those satisfying the safety valve function requirement, shall be OPERABLE with the specified lift settings:

| <u>Number of Valves</u> | <u>Function</u> | <u>Setpoint* (psig)</u> |
|-------------------------|-----------------|-------------------------|
| 7                       | Safety          | 1165 (+)0(-)2%          |
| 5                       | Safety          | 1180 (+)0(-)2%          |
| 4                       | Safety          | 1190 (+)0(-)2%          |
| 1                       | Relief          | 1103 ± 15 psig          |
| 8                       | Relief          | 1113 ± 15 psig          |
| 7                       | Relief          | 1123 ± 15 psig          |

The acoustic monitor for each OPERABLE valve shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

#### ACTION:

- a. With the safety and/or relief valve function of one or more of the above required safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With one or more safety/relief valves stuck open, close the stuck open safety/relief valve(s); if suppression pool average water temperature is 105°F or greater, place the reactor mode switch in the Shutdown position.
- c. With one or more safety/relief valve acoustic monitors inoperable, restore the inoperable monitor(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

\*The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures.



## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS

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4.4.2.1.1 The acoustic monitor for each safety/relief valve shall be demonstrated OPERABLE by performance of a:

- a. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
- b. CHANNEL CALIBRATION at least once per 18 months.\*

4.4.2.1.2 The relief valve function pressure actuation instrumentation shall be demonstrated OPERABLE by performance of a:

- a. CHANNEL FUNCTIONAL TEST, including calibration of the trip unit set-point, at least once per 31 days.
- b. CHANNEL CALIBRATION, LOGIC SYSTEM FUNCTIONAL TEST and simulated automatic operation of the entire system at least once per 18 months.

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\*The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the test.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-47

GULF STATES UTILITIES COMPANY

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated February 18, 1988 as revised March 22, 1988, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The proposed amendment would modify Technical Specification (TS) 3.4.2.1. to change the setpoint tolerance of the safety valve function of the safety/relief valves from  $\pm 1\%$  to (+)0(-)2% of the set pressure. This proposed change would make the tolerance consistent with Section 5.2.2.10 of the licensee's safety analysis report and the vendor's design specification. The March 22, 1988 submittal corrected an editorial error and did not alter the staff's proposed no significant hazards consideration issued on April 6, 1988.

2.0 EVALUATION

The 16 safety/relief valves (S/RVs) are mounted on the 4 main steam lines and provide overpressure protection for the reactor coolant pressure boundary. The current River Bend TS 3.4.2.1 requires that the setpoint tolerance for the safety function of the S/RVs be within  $\pm 1\%$  of the setpoint. Section 5.2.2.10 of the updated safety analysis report (USAR), on the other hand, states that the valves must operate within +0 percent to -2 percent of the nameplate set pressure. The licensee's February 18, 1988 submittal states that the correct set pressure tolerance is as stated in the USAR which is also consistent with the reactor vendor's design specification. Accordingly, the licensee has requested that the TSs be modified to agree with the tolerance stated in the USAR. The licensee's submittal states that two of the 16 S/RVs have been declared inoperable because the setpoints do not meet the  $\pm 1\%$  tolerance; however, they do meet the +0 percent to -2 percent tolerance. The remaining 14 valves are operable and TS 3.4.2.1 allows continued operation in this configuration.

The licensee's proposed change would reduce the maximum allowable safety function setpoint by lowering the setpoint tolerance by 1 percent. This change is in the conservative direction because the safety valve function will occur at a lower system pressure.

Based on the above considerations, the staff concludes that the proposed change in the S/RV setpoint tolerances is acceptable.

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### 3.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 10, 1988

Principal Contributor: W. Paulson