December 9, 1987

Docket No. 50-458

Mr. James C. Deddens
Senior Vice President, (RBNG)
Gulf States Utilities
P. O. Box 220
St. Francisville, LA 70775
ATTN: Nuclear Licensing

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Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-47 (TAC NO. 65728)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 17 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated June 18, 1987 as supplemented July 31, 1987 and revised October 8, 1987.

The amendment revised Section 6.0, Administrative Controls, of the TSs. The changes include revisions to the River Bend Nuclear Group Organization, the River Bend Station Organization, and the composition of both the Facility Review Committee and the Nuclear Review Board. In addition, the TSs have been changed to meet the Commission Policy Statement on Engineering Expertise on Shift.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 17 to License No. NPF-47
2. Safety Evaluation

cc w/enclosures:
See next page

LTR NAME: (River Bend Amndmt)

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11/24/87	11/25/87	12/2/87	12/9/87

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 17
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by Gulf States Utilities Company, dated June 18, 1987 as supplemented July 31, 1987 and revised October 8, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 17 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 9, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 17

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.

REMOVE PAGES	INSERT PAGES
6-1	6-1
6-2	6-2
6-3	6-3
6-4	6-4
6-6	6-6
6-7	6-7
6-8	6-8
6-9	6-9
6-10	6-10
6-15	6-15
6-19	6-19

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Supervisor (or during his absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Senior Vice President - River Bend Nuclear Group, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for unit management and technical support shall be as shown in Chapter 13 of the current Safety Analysis Report.

UNIT STAFF

6.2.2 The unit organization shall be as shown in Chapter 13 of the current Safety Analysis Report and:

a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.2-1;

b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in OPERATIONAL CONDITION 1, 2 or 3, at least one licensed Senior Operator shall be in the control room;

c. A Radiation Protection Technician* shall be on site when fuel is in the reactor;

d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;

e. **A site fire brigade of at least five members shall be maintained on site at all times*. The fire brigade shall not include the Shift Supervisor, the Shift Technical Advisor, the Control Operating Foreman,** nor the two other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency; and

*The Radiation Protection Technician and fire brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

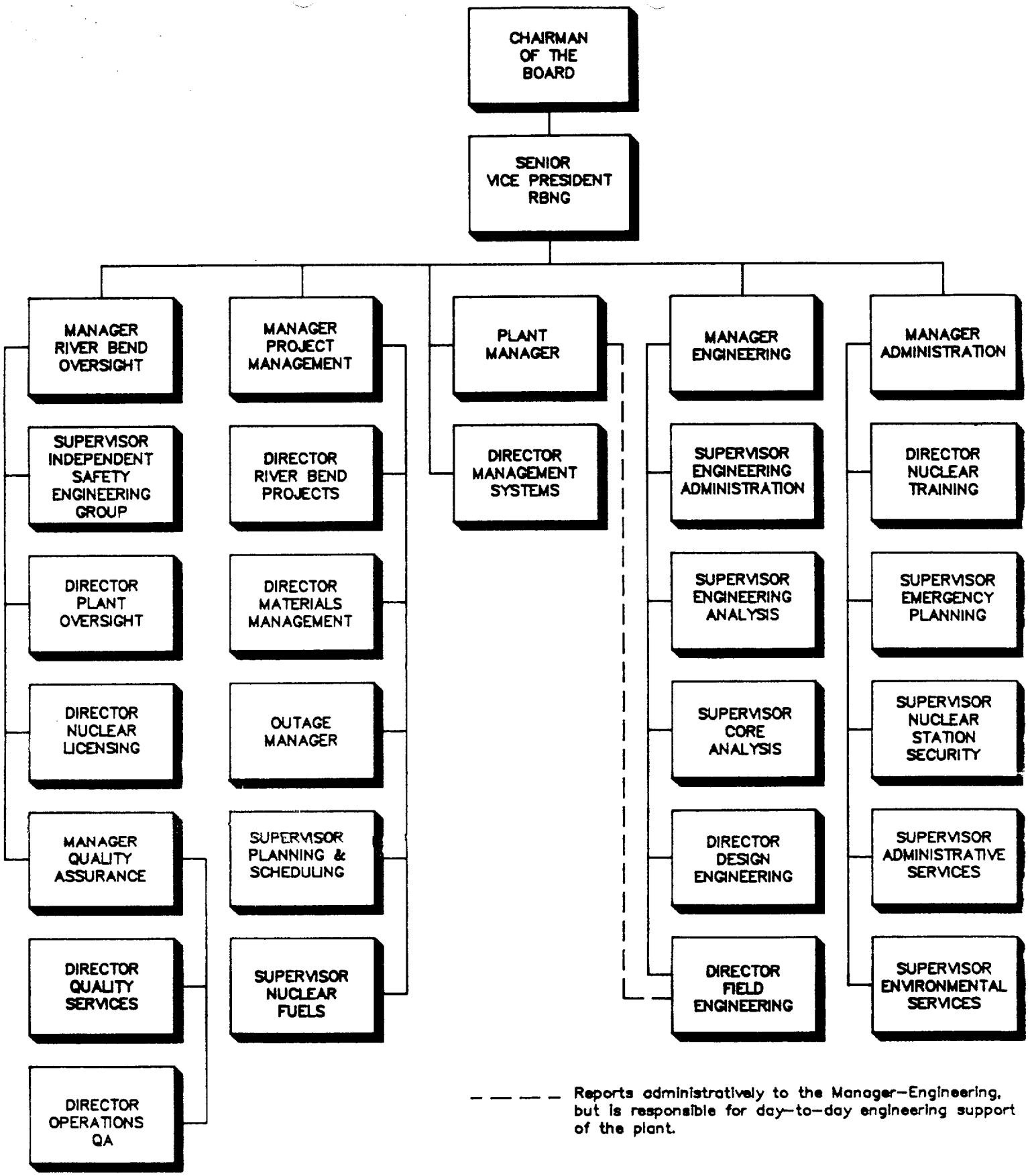

FIGURE 6.2.1–1
RIVER BEND STATION ORGANIZATION

TABLE 6.2.2-1

MINIMUM SHIFT CREW COMPOSITION

SINGLE UNIT FACILITY

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITION 1, 2, or 3	CONDITION 4 or 5
SS	1	1
SRO	1	None
RO	2	1
AO	2	1
STA*	1	None

TABLE NOTATION

SS - Shift Supervisor with a Senior Operator license on Unit 1.
SRO - Individual with a Senior Operator license on Unit 1.
RO - Individual with an Operator license on Unit 1.
AO - Auxiliary operator
STA - Shift Technical Advisor

*The Shift Technical Advisor (STA) position may be filled by an on-shift Shift Supervisor (SS) or Senior Reactor Operator (SRO) provided the individual meets the STA qualifications of Specification 6.2.4 and five (5) licensed operators are on shift.

The shift crew composition may be one less than the minimum requirements of Table 6.2.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the control room while the unit is in OPERATIONAL CONDITION 1, 2 or 3, an individual with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Shift Supervisor from the control room while the unit is in OPERATIONAL CONDITION 4 or 5, an individual with a valid Senior Operator license or Operator license shall be designated to assume the control room command function.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the Director-Radiological Programs who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Director - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 FACILITY REVIEW COMMITTEE (FRC)

FUNCTION

6.5.1.1 The FRC shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The FRC shall be composed of the:

Chairman:	Assistant Plant Manager-Technical Services
Member:	Assistant Plant Manager-Operations and Radwaste
Member:	Assistant Plant Manager-Maintenance
Member:	Operations Supervisor
Member:	Director-Radiological Program
Member:	Reactor Engineering Supervisor
Member:	Control Systems Supervisor
Member:	Chemistry Supervisor
Member:	Process Systems Supervisor

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the FRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in FRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The FRC shall meet at least once per calendar month and as convened by the FRC Chairman or his designated alternate.

RECORDS

6.5.1.8 The FRC shall maintain written minutes of each FRC meeting that, at a minimum, document the results of all FRC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Plant Manager and the NRB.

6.5.2 TECHNICAL REVIEW AND CONTROL

6.5.2.1 Each procedure and program required by Specification 6.8 and other procedures that affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group that prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group that prepared the procedure. Each such procedure and program, or changes thereto, shall be approved, prior to implementation, by the Plant Manager or one of the Assistant Plant Managers or the Director - Radiological Programs, with the exception of the Emergency Plan and implementing procedures which shall be approved by the Manager - Administration, Plant Manager and Senior Vice President - RBNG.

6.5.2.2 Individuals responsible for reviews performed in accordance with Section 6.5.2.1 shall be members of River Bend Nuclear Group supervisory staff, and the reviews shall be performed in accordance with administrative procedures. Each such review shall include a determination of whether or not additional, cross-disciplinary review is necessary and a verification that the proposed actions do not constitute an unreviewed safety question. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

6.5.2.3 The station security program and implementing procedures shall be reviewed at least once per 12 months, and recommended changes approved in accordance with Specification 6.5.2.1.

6.5.2.4 The station emergency plan and implementing procedures and recommended changes shall be approved in accordance with Specification 6.5.2.1.

6.5.2.5 The station fire protection plan and implementing procedures shall be reviewed at least once per 12 months, and recommended changes approved in accordance with Specification 6.5.2.1.

6.5.2.6 Records documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.5 shall be maintained.

6.5.3 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

6.5.3.1 The NRB shall function to provide independent review and audit of designated activities in the areas of:

PROCEDURES AND PROGRAMS (Continued)

2. Integrated leak test requirements for each system, at refueling cycle intervals or more frequently.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

c. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents and containment atmosphere samples, under accident conditions. The program shall include the following:

1. Training of personnel,
2. Procedures for sampling and analysis, and
3. Provisions for maintenance of sampling and analysis equipment.

d. Biofouling Prevention and Detection

A program, approved by the NRC Staff prior to introduction of river water to the systems, which will include the procedures to prevent biofouling of safety-related equipment, to assure detection of Corbicula in the intake embayment and the Mississippi River at the River Bend Station site, and to monitor and survey safety-related equipment to detect biofouling. Changes to this program will be submitted to and approved by the NRC (both the Region and NRR) prior to implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the U.S. Nuclear Regulatory Commission, Document Control Desk, Washington, DC 20555, with a copy to the Regional Office of the NRC and a copy to the NRC Resident Inspector, unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an Operating License, (2) amendment to

SEMIANNUAL EFFLUENT RELEASE REPORT (Continued)

to MEMBERS OF THE PUBLIC due to their activities inside the SITE BOUNDARY (Figure 5.1.3) during the report period. All assumptions used in making these assessments (i.e., specific activity, exposure time and location) shall be included in these reports. The assessment of radiation doses shall be performed in accordance with the methodology and parameters of the OFFSITE DOSE CALCULATION MANUAL (ODCM).

The Semiannual Radioactive Effluent Release Report to be submitted 60 days after January 1 of each year shall also include an assessment of radiation doses to the likely most-exposed MEMBER OF THE PUBLIC from reactor releases and other nearby uranium fuel cycle sources (including doses from primary effluent pathways and direct radiation) for the previous calendar year to show conformance with 40 CFR Part 190, Environmental Radiation Protection Standards for Nuclear Power Operation. Acceptable methods for calculating the dose contribution from liquid and gaseous effluents are given in Regulatory Guide 1.109, Rev. 1, October 1977.

The Semiannual Radioactive Effluent Release Reports shall include a list and description of unplanned releases from the site to UNRESTRICTED AREAS of radioactive materials in gaseous and liquid effluents made during the reporting period.

The Semiannual Radioactive Effluent Release Reports shall include any changes made during the reporting period to the PROCESS CONTROL PROGRAM (PCP) and to the ODCM, as well as a listing of new locations for dose calculations and/or environmental monitoring identified by the land use census pursuant to Specification 3.12.2

SPECIAL REPORTS

6.9.2 Special reports shall be submitted in the following manner:

a. Special reports shall be submitted to the U.S. Nuclear Regulatory Commission, Document Control Desk, Washington, DC 20555, with a copy to the Regional Office of the NRC and a copy to the NRC Resident Inspector, within the time period specified for each report.

b. Special reports in regard to Corbicula will be submitted to the NRC within 30 days of identification of infestation. In accordance with the settlement agreement dated October 10, 1984, these reports shall describe the level of infestation, affected systems and measures taken to prevent further infestation.

6.10 RECORD RETENTION

6.10.1 In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.2 The following records shall be retained for at least 5 years:

a. Records and logs of unit operation covering time interval at each power level.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-47

GULF STATES UTILITIES COMPANY

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated June 18, 1987, as supplemented July 31, 1987 and revised October 8, 1987, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The proposed amendment would revise Section 6.0, Administration Controls, of the Technical Specifications. The proposed changes would revise the River Bend Nuclear Group Organization, the River Bend Station Organization, and the composition of both the Facility Review Committee and the Nuclear Review Board. In addition, a change is proposed to meet the Commission Policy Statement on Engineering Expertise on Shift.

In the June 18, 1987 application, the licensee proposed to delete Figure 6.2.1-1, River Bend Nuclear Group Organization, and Figure 6.2.2-1, River Bend Station Organization on the basis that the offsite organization for unit management and technical support is identified in Chapter 13 of the Safety Analysis Report. By letter dated August 25, 1987, the staff indicated that the issue of organizational charts in the TSs is being considered as part of the Technical Specification Improvement Program. In the meantime, the staff requires that organizational charts remain in the TSs. The licensee's October 8, 1987 submittal included these charts. In addition, this submittal provided clarification of the organizational changes and also clarification of the qualifications of the shift technical advisor (STA) for the dual role position of STA/Shift Supervisor or Senior Reactor Operator (SRO). The October 8, 1987 submittal also proposed that the Nuclear Review Board be comprised of a total of 12 members, the current total membership, an increase from the 11 proposed in the June 18, 1987 application. These changes are included in the following evaluation.

2.0 EVALUATION

2.1 River Bend Nuclear Group Organization

The licensee has revised the offsite organization responsible for supporting the operation of the River Bend Station, Unit 1. The positions of Vice President Safety and Environment, Vice President River Bend Nuclear Group, Manager-Project Control, Manager-Engineering, Nuclear Fuels and Licensing, Director-Nuclear Fuels Design and Safety Analysis, Director-Nuclear Plant Engineering, and Director-Nuclear

Procurement and Contracts have been deleted. The positions of Manager-River Bend Oversight, Manager-Engineering, Director-Design Engineering, Supervisor-Planning and Scheduling, Supervisor-Nuclear Fuels, Supervisor-Engineering Analysis, and Director-Materials Management have been added.

The titles of Manager-Project Planning has been changed to Manager-Project Management, Supervisor Administrative Support has been changed to Supervisor Support Services, Plant Security Supervisor has been changed to Supervisor-Nuclear Station Security, and Supervisor-In-Core Fuels has been changed to Supervisor-Core Analysis. Additionally, several functions have been reassigned within the revised organization.

These changes are shown in revised Figure 6.2.1-1.

The staff finds these requested changes acceptable as they meet the acceptance criteria of Section 13.1.1 of NUREG-0800, Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plant, (the Standard Review Plan).

2.2 River Bend Station Organization

The utility has revised the organization responsible for the operation and maintenance of the River Bend Station. The position of Superintendent-Start-Up and Test has been deleted. The titles of General Operations Supervisor and Supervisor-Radiological Programs have been changed to Operations Supervisor and Director-Radiological Programs respectively. The new position of Outage Manager who will report to the Plant Manager has been created.

The position of Assistant Plant Manager-Maintenance and Material has been changed to Assistant Plant Manager-Maintenance with the functions related to control, inventory and procurement of materials assigned to the new position of Director-Materials.

The position of Assistant Plant Manager-Operations, Radwaste and Chemistry has been changed to Assistant Plant Manager-Operations and Radwaste, with the responsibility for the chemistry program being transferred under the Chemistry Supervisor to the Assistant Plant Manager-Technical Services.

These changes are shown in revised Figure 6.2.2-1.

The staff finds these requested changes acceptable as they meet the acceptance criteria of Section 13.1.2-13.1.3 of NUREG-0800, the Standard Review Plan.

2.3 Shift Technical Advisor (Section 6.2.4)

The licensee has revised this section to allow for the dual role position of STA/Shift Supervisor or SRO by stating that when the STA is used in the dual role that individual will meet the education and training requirements of the Commission Policy Statement on Engineering Expertise on Shift.

The staff finds this requested change acceptable as it meets the Commission Policy Statement on Engineering Expertise on Shift.

2.4 Facility Review Committee (FRC) (Section 6.5.1)

The licensee has revised the composition of the FRC (Section 6.5.1.2) to reflect the revised organization. This has resulted in the membership being increased by two.

The staff finds this requested change acceptable as the quorum requirement represents a majority of the FRC and the change meets the acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan.

2.5 Nuclear Review Board (NRB) (Section 6.5.3)

The licensee has revised the composition (Section 6.5.3.2) of the NRB to reflect the revised organization.

The staff finds this requested change acceptable as the quorum requirement continues to be a majority of the committee and it meets the acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan.

2.6 Editorial Changes

The licensee has made numerous changes in titles throughout Section 6 of the Technical Specifications to reflect the revised organization.

We have reviewed these changes and found they are acceptable as they reflect the revised organization.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting or administrative procedures or requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding.

Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. The staff therefore concludes that the proposed changes are acceptable, and they are hereby incorporated into the River Bend Unit 1 Technical Specifications.

Principal Contributors: F. Allenspach

Dated: December 9, 1987