

August 29, 1988

Docket No. 50-458

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. 26 TO FACILITY
OPERATING LICENSE NO. NPF-47 (TAC NO. 68422)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 26 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 31, 1988.

The amendment adds action requirements to TS 3.1.3.1 in the event that the scram discharge volume vent and/or drain valves become inoperable.

A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 26 to License No. NPF-47
2. Safety Evaluation

cc w/enclosures:
See next page

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DOCUMENT NAME: RIVER BEND AMEND TAC 68422

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script that reads "Walter A. Paulson".

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

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See next page

Mr. James C. Deddens
Gulf States Utilities Company

River Bend Nuclear Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Gulf States Utilities Company (the licensee) dated May 31, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

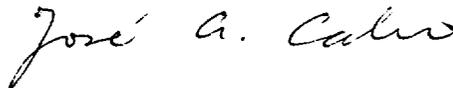
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 26 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 29, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 26

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages provided to maintain document completeness.

REMOVE PAGES

3/4 1-4
3/4 1-5

INSERT PAGES

3/4 1-4
3/4 1-5 (No change, reissued
due to repagination)

REACTIVITY CONTROL SYSTEMS

3/4.1.3 CONTROL RODS

CONTROL ROD OPERABILITY

LIMITING CONDITION FOR OPERATION

3.1.3.1 All control rods shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

ACTION:

- a. With one control rod inoperable, due to being immovable as a result of excessive friction or mechanical interference, or known to be untrippable:
 1. Within one hour:
 - a) Verify that the inoperable control rod, if withdrawn, is separated from all other inoperable control rods by at least two control cells in all directions.
 - b) Disarm the associated directional control valves* either:
 - 1) Electrically, or
 - 2) Hydraulically by closing the drive water and exhaust water isolation valves.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
 2. Restore the inoperable control rod to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With one or more control rods trippable but inoperable for causes other than addressed in ACTION a, above:
 1. If the inoperable control rod(s) is withdrawn, within one hour:
 - a) Verify that the inoperable withdrawn control rod(s) is separated from all other inoperable withdrawn control rods by at least two control cells in all directions, and
 - b) Demonstrate the insertion capability of the inoperable withdrawn control rod(s) by inserting the control rod(s) at least one notch by drive water pressure within the normal operating range**.

*May be rearmed intermittently, under administrative control, to permit testing associated with restoring the control rod to OPERABLE status.

**The inoperable control rod may then be withdrawn to a position no further withdrawn than its position when found to be inoperable.

REACTIVITY CONTROL SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

Otherwise, insert the inoperable withdrawn control rod(s) and disarm the associated directional control valves* either:

- a) Electrically, or
 - b) Hydraulically by closing the drive water and exhaust water isolation valves.
2. If the inoperable control rod(s) is inserted, within one hour disarm the associated directional control valves* either:
- a) Electrically, or
 - b) Hydraulically by closing the drive water and exhaust water isolation valves.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.

3. The provisions of Specification 3.0.4 are not applicable.
- c. With more than 8 control rods inoperable, be in at least HOT SHUTDOWN within 12 hours.
 - d. With one scram discharge volume vent valve and/or one scram discharge volume drain valve inoperable and open, restore the inoperable valve(s) to OPERABLE status within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours.
 - e. With two scram discharge volume vent valves and/or two scram discharge volume drain valves inoperable and open, restore one valve in the vent line and one valve in the drain line to OPERABLE status within 8 hours and restore all valves to OPERABLE status within the next 16 hours or close at least one vent valve and one drain valve and be in at least HOT SHUTDOWN within the next 12 hours.
 - f. With any scram discharge volume vent valve(s) and/or any scram discharge volume drain valve(s) inoperable and closed except when required by ACTION statement e. above, restore all valves to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The scram discharge volume drain and vent valves shall be demonstrated OPERABLE by:

- a. At least once per 31 days verifying each valve to be open,** and

*May be rearmed intermittently, under administrative control, to permit testing associated with restoring the control rod to OPERABLE status.

**These valves may be closed intermittently for testing under administrative controls.

REACTIVITY CONTROL SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 92 days cycling each valve through at least one complete cycle of full travel.

4.1.3.1.2 When above the low power setpoint of the RPCS, all withdrawn control rods not required to have their directional control valves disarmed electrically or hydraulically shall be demonstrated OPERABLE by moving each control rod at least one notch:

- a. At least once per 7 days, and
- b. At least once per 24 hours when any control rod is immovable as a result of excessive friction or mechanical interference.

4.1.3.1.3 All control rods shall be demonstrated OPERABLE by performance of Surveillance Requirements 4.1.3.2, 4.1.3.3, 4.1.3.4 and 4.1.3.5.

4.1.3.1.4 The scram discharge volume shall be determined OPERABLE by demonstrating the scram discharge volume drain and vent valves OPERABLE, at least once per 18 months, by verifying that the drain and vent valves:

- a. Close within 30 seconds after receipt of a signal for control rods to scram, and
- b. Open when the scram signal is reset.

REACTIVITY CONTROL SYSTEMS

CONTROL ROD MAXIMUM SCRAM INSERTION TIMES

LIMITING CONDITION FOR OPERATION

3.1.3.2 The maximum scram insertion time of each control rod from the fully withdrawn position, based on de-energization of the scram pilot valve solenoids as time zero, shall not exceed the following limits:

Reactor Vessel Dome Pressure (psig)*	Maximum Insertion Times to Notch Position (Seconds)		
	43	29	13
950	0.31	0.81	1.44
1050	0.32	0.86	1.57

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

ACTION:

- a. With the maximum scram insertion time of one or more control rods exceeding the maximum scram insertion time limits of Specification 3.1.3.2 as determined by Surveillance Requirement 4.1.3.2.a or b, operation may continue provided that:
 1. For all "slow" control rods, i.e., those which exceed the limits of Specification 3.1.3.2, the individual scram insertion times do not exceed the following limits:

Reactor Vessel Dome Pressure (psig)*	Maximum Insertion Times to Notch Position (Seconds)		
	43	29	13
950	0.38	1.09	2.09
1050	0.39	1.14	2.22

2. For "fast" control rods, i.e., those which satisfy the limits of Specification 3.1.3.2, the average scram insertion times do not exceed the following limits:

Reactor Vessel Dome Pressure (psig)*	Maximum Average Insertion Times to Notch Position (Seconds)		
	43	29	13
950	0.30	0.78	1.40
1050	0.31	0.84	1.53

3. The sum of "fast" control rods with individual scram insertion times in excess of the limits of ACTION a.2 and of "slow" control rods does not exceed 5.

*For intermediate reactor vessel dome pressure, the scram time criterion is determined by linear interpolation at each notch position.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. NPF-47

GULF STATES UTILITIES COMPANY

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated May 31, 1988, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The proposed amendment would add Action Statements to Technical Specification (TS) 3.1.3.1 in the event that the scram discharge volume (SDV) vent and drain valves become inoperable. Current TSs 4.1.3.1.1 and 4.1.3.1.4 require that surveillance testing of these vent and drain valves be performed to demonstrate the operability of the SDV; however, there are currently no Action Statements provided should any of these valves become inoperable.

The proposed TSs are similar to those previously approved in the staff's October 17, 1986 safety evaluation supporting the issuance of Amendment No. 21 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit No. 1.

2.0 EVALUATION

The purpose of the SDV is to serve as a collection volume for water displaced by the control rod drive pistons during a scram. During normal operation, the SDV vent and drain valves remain open to allow operational leakage from the scram system to drain from the SDV to the containment equipment drain sumps. The vent and drain lines have redundant valves on each of these lines. These valves automatically close upon the receipt of a scram signal and isolate the SDV thereby preventing the discharged reactor coolant from being released to the containment during a scram. In addition, these valves close in the event of loss of air and/or loss of electrical power.

To assure that there will be sufficient capability in the SDV to accommodate the displaced water, the reactor is automatically scrammed if the water level in the SDV exceeds the level setpoint. This setpoint is selected so that there will be adequate capacity for the discharged reactor coolant. High water level in the SDV is detected with both float switches and level transmitters.

Proposed Action Statement d. of TS 3.1.3.1 gives the required action if one vent valve and/or one drain valve is found to be inoperable and open. As previously stated, during normal operation, the SDV is vented and drained to the containment equipment sump. Each vent and drain line

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contains two valves in series which close on a scram signal. The proposed Action d. allows 24 hours of operation with the inoperable valve(s). If the inoperable valve(s) cannot be made operable within 24 hours, the plant shall be placed in at least hot shutdown within the next 12 hours. This proposed Action Statement conforms to that of the Grand Gulf, Perry, and Clinton facilities, which are also BWR-6, Mark III plants. The staff concludes that proposed Action Statement d. is acceptable because in the event of a scram, the remaining operable vent and drain valves can still be expected to perform their function of isolating the SDV.

Proposed Action Statement e. of TS 3.1.3.1 gives the required action if two SDV vent and/or drain valves are found to be inoperable and open. In the event of a scram while operating in this condition, reactor coolant would be released to the containment equipment drain sumps. This effluent would be contained within primary containment. Adequate reactor vessel makeup water would be available from the condensate and feedwater system or the high pressure core spray. The proposed action includes requiring that one vent valve and one drain valve be made operable (permit closure on a scram) within eight hours. This is more restrictive than the 24 hours allowed for operation with one vent valve and/or one drain valve inoperable and open as discussed in proposed Action Statement d. The eight hours allows time for the restoration of the valve operability. The action further requires restoring all valves to operable status in the next 16 hours or be in at least hot shutdown within the next 12 hours. This additional 16 hour restoration time for the remaining inoperable vent valve and/or drain valve is consistent with proposed Action Statement d. The required action is that all vent valves and drain valves be made operable within 24 hours. The staff concludes that proposed Action Statement e. is acceptable.

Proposed Action Statement f. of TS 3.1.3.1 provides the required actions if any SDV vent and/or drain valve is found to be inoperable and closed. The action is to restore all valves to the operable status within 8 hours or be in at least hot shutdown within the next 12 hours. The staff finds that is acceptable because if a scram does occur, the inoperable valve(s) is in the preferred closed position and therefore the safe shutdown capability of the plant would not be adversely affected. The staff concludes that the proposed Action Statement is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public

comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 29, 1988

Principal Contributor: W. Paulson