

April 5, 1988

Docket No. 50-458

Mr. James C. Deddens
Senior Vice President, (RBNG)
Gulf States Utilities
P. O. Box 220
St. Francisville, LA 70775
ATTN: Nuclear Licensing

Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1 - AMENDMENT NO. ¹⁹ TO FACILITY
OPERATING LICENSE NO. NPF-47 (TAC NO. 65729)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. ¹⁹ to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated June 18, 1987 as clarified by letter dated March 11, 1988.

The amendment revises the Technical Specification to increase the leak detection setpoints and allowable values for the reactor water cleanup system heat exchanger room temperature and decreases the reactor core isolation cooling system isolation trip setpoint and allowable values for the residual heat removal steam line flow-high.

A copy of our Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. ¹⁹ to License No. NPF-47
2. Safety Evaluation

cc w/enclosures:

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DOCUMENT NAME: RIVER BEND AMENDMENT TAC 65729

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JKR



Docket No. 50-458

UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555
April 5, 1988

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Mr. James C. Deddens
Gulf States Utilities Company

River Bend Nuclear Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 19
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Gulf States Utilities Company (the licensee) dated June 18, 1987, as supplemented March 11, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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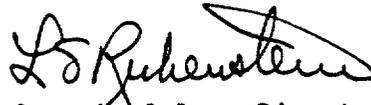
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1⁹ and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 5, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 19

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.

REMOVE PAGES

3/4 3-20
3/4 3-21
3/4 3-22

INSERT PAGES

3/4 3-20
3/4 3-21
3/4 3-22

TABLE 3.3.2-2
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low, Level 2	≥ -43 inches*	≥ -47 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Containment Purge Isolation Radiation - High	≤ 1.3 R/hr	≤ 1.57 R/hr
2. <u>MAIN STEAM LINE ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -143 inches*	≥ -147 inches
b. Main Steam Line Radiation - High	≤ 3.0 x full power background	≤ 3.6 x full power background
c. Main Steam Line Pressure - Low	≥ 849 psig	≥ 837 psig
d. Main Steam Line Flow - High		
1. Line A	< 146 psid	< 151 psid
2. Line B	< 156 psid	< 161 psid
3. Line C	< 153 psid	< 158 psid
4. Line D	< 164 psid	< 169 psid
e. Condenser Vacuum - Low	≥ 8.5 inches Hg. vacuum	≥ 7.6 inches Hg. vacuum
f. Main Steam Line Tunnel Temperature - High	$\leq 135^\circ\text{F}$	$\leq 142.5^\circ\text{F}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 51^\circ\text{F}$	$\leq 55^\circ\text{F}$

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
2. <u>MAIN STEAM LINE ISOLATION (Cont'd)</u>		
h. Main Steam Line Area Temperature - High (Turbine Building)		
1. Main Steam Tunnel Area (El. 95')	< 142°F	< 145.3°F
2. Main Steam Tunnel Area (El. 114')	< 142°F	< 145.3°F
3. Main Steam Line Turbine Shield Wall	< 102°F	< 106°F
4. MSL Moisture Separator and Reheater Area	< 126°F	< 130°F
3. <u>SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low Level 2	≥ - 43 inches*	≥ - 47 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Fuel Building Ventilation Exhaust Radiation - High		
IRMS*RE5A	< 1.82 x 10 ³ μCi/sec	< 2.18 x 10 ³ μCi/sec
IRMS*RE5B	≤ 5.88 x 10 ⁻⁴ μCi/cc	≤ 7.05 x 10 ⁻⁴ μCi/cc
d. Reactor Building Annulus Ventilation Exhaust Radiation - High	≤ 4.32 x 10 ⁻⁵ μCi/cc	≤ 5.19 x 10 ⁻⁵ μCi/cc
4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>		
a. Δ Flow - High	≤ 55 gpm	≤ 62.1 gpm
b. Δ Flow Timer	≤ 45 seconds	≤ 47 seconds
c. Equipment Area Temperature - High		
1. Heat Exchanger Room	< 104.5°F	< 107.5°F
2. Pump Rooms A & B	< 165°F	< 169.5°F
3. Valve Nest Room	< 110°F	< 114.5°F
4. Demineralizer Rooms 1 and 2	< 110°F	< 114.5°F
5. Receiving Tank Room	< 110°F	< 114.5°F

RIVER BEND - UNIT 1

3/4 3-20

AMENDMENT NO. 1, 19

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>4. REACTOR WATER CLEANUP SYSTEM ISOLATION (Cont'd)</u>		
d. Equipment Area Δ Temperature - High		
1. Heat Exchanger Room	$< 39^{\circ}\text{F}$	$< 42.5^{\circ}\text{F}$
2. Pump Rooms A and B	$< 78^{\circ}\text{F}$	$< 82^{\circ}\text{F}$
3. Valve Nest Room	$< 46^{\circ}\text{F}$	$< 49.5^{\circ}\text{F}$
4. Demineralizer Rooms 1 and 2	$< 46^{\circ}\text{F}$	$< 49.5^{\circ}\text{F}$
5. Receiving Tank Room	$< 46^{\circ}\text{F}$	$< 49.5^{\circ}\text{F}$
e. Reactor Vessel Water Level - Low Low Level 2	$\geq - 43$ inches*	$\geq - 47$ inches
f. Main Steam Line Tunnel Ambient Temperature - High	$\leq 135^{\circ}\text{F}$	$\leq 142.5^{\circ}\text{F}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 51^{\circ}\text{F}$	$\leq 55^{\circ}\text{F}$
h. SLCS Initiation	NA	NA
<u>5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>		
a. RCIC Steam Line Flow - High	$\leq 127'' \text{H}_2\text{O}^{**}$	$\leq 135.5'' \text{H}_2\text{O}^{**}$
b. RCIC Steam Line Flow - High Timer	≥ 3 seconds	≤ 13 seconds
c. RCIC Steam Supply Pressure - Low	≥ 60 psig	≥ 55 psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10 psig	≤ 20 psig
e. RCIC Equipment Room Ambient Temperature - High	$\leq 182^{\circ}\text{F}$	$\leq 186.4^{\circ}\text{F}$
f. RCIC Equipment Room Δ Temperature - High	$\leq 96^{\circ}\text{F}$	$\leq 99^{\circ}\text{F}$

RIVER BEND - UNIT 1

3/4 3-21

AMENDMENT NO. 11, 19

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u> (Cont'd)		
g. Main Steam Line Tunnel Ambient Temperature - High	$\leq 135^{\circ}\text{F}$	$\leq 142.5^{\circ}\text{F}$
h. Main Steam Line Tunnel Δ Temperature - High	$\leq 51^{\circ}\text{F}$	$\leq 55^{\circ}\text{F}$
i. Main Steam Line Tunnel Temperature Timer	0 seconds	NA
j. RHR Equipment Room Ambient Temperature - High	$\leq 117^{\circ}\text{F}$	$\leq 121.1^{\circ}\text{F}$
k. RHR Equipment Room Δ Temperature - High	$\leq 29^{\circ}\text{F}$	$\leq 33.6^{\circ}\text{F}$
l. RHR/RCIC Steam Line Flow - High	$\leq 60.7'' \text{H}_2\text{O}$	$\leq 64.2'' \text{H}_2\text{O}$
m. Drywell Pressure - High	$\leq 1.68 \text{ psig}$	$\leq 1.88 \text{ psig}$
n. Manual Initiation	NA	NA
6. <u>RHR SYSTEM ISOLATION</u>		
a. RHR Equipment Area Ambient Temperature - High	$\leq 117^{\circ}\text{F}$	$\leq 121.1^{\circ}\text{F}$
b. RHR Equipment Area Δ Temperature - High	$\leq 29^{\circ}\text{F}$	$\leq 33.6^{\circ}\text{F}$
c. Reactor Vessel Water Level - Low Level 3	$\geq 9.7 \text{ inches}^*$	$\geq 8.7 \text{ inches}$
d. Reactor Vessel Water Level - Low Low Level 1	$\geq - 143 \text{ inches}^*$	$\geq - 147 \text{ inches}$



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. NPF-47

GULF STATES UTILITIES COMPANY

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated June 18, 1987, the Gulf States Utilities Company, the licensee for the River Bend Station, Unit 1, proposed to change the plant Technical Specifications (TS) Table 3.3.2-2 for leak detection setpoints for the reactor water cleanup area temperatures (items 4.c.1 and 4.d.1), and the RCIC/RHR steam line high flow isolation (item 5.1). The licensee provided additional clarification in a letter dated March 11, 1988. The March 11, 1988 letter was in response to a request from the staff to document the clarification and did not influence or change the basis for the proposed no significant hazards consideration issued on August 12, 1987.

The specific changes are as follows:

Reactor Water Cleanup (RWCU) Area Temperature - The licensee proposed to change the RWCU heat exchanger room ambient high temperature trip setpoint for RWCU isolation, Item 4.c.1 from $\leq 98.5^{\circ}\text{F}$ to $\leq 104.5^{\circ}\text{F}$ and the allowable value from $\leq 101.5^{\circ}\text{F}$ to $\leq 107.5^{\circ}\text{F}$. The licensee also proposed to change the differential temperature trip setpoint in the above area, Item 4.d.1, from $\leq 33^{\circ}\text{F}$ to $\leq 39^{\circ}\text{F}$ and the allowable value from $\leq 36.5^{\circ}\text{F}$ to $\leq 42.5^{\circ}\text{F}$. The licensee stated that the proposed changes are required to avoid unnecessary engineered safety feature (ESF) isolation signals due to restricting RWCU heat-exchanger room ambient and differential temperature setpoints.

RCIC/RHR Steam Line High Flow Isolation - The licensee proposed to change the RCIC/RHR steam line high flow isolation initial trip setpoint, Item 5.1, from $\leq 156'' \text{H}_2\text{O}$ to $\leq 60.7'' \text{H}_2\text{O}$ and the allowable value from $\leq 164.5'' \text{H}_2\text{O}$ to $\leq 64.2'' \text{H}_2\text{O}$. The licensee stated that the above change is necessary as a result of the review of the RCIC/RHR steam line high flow setpoint.

2.0 EVALUATION

Reactor Water Cleanup Area Temperature - The licensee stated that the original leak detection temperature setpoints were based on a predicted temperature in the containment area and assuming a steam leakage rate of 25 gpm. The proposed setpoints have the same analytical basis as the original setpoints with the exception that the initial temperature is based on actual temperature data rather than the predicted value. The applied setpoint methodology, utilizing the actual temperature data,

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accounts for instrument inaccuracies and drift to ensure the margin of safety is maintained. The licensee also stated that a review was performed to ensure that the existing design of the equipment and structures enveloped the higher normal operating temperature and that the qualified life of the equipments were revised as necessary.

The Office for Analysis and Evaluation of Operational Data (AEOD) issued a report numbered AEOD/E705 on March 31, 1987, entitled "RWCU System Automatic Isolation and Safety Consideration." This report documents a review of the Licensee Event Reports (LERs) from January 1984 through October 1986 concerning RWCU isolations and has made the following conclusions. Of all of the RWCU isolations; 74% were due to spurious signals. Slightly less than half of the isolations were initiated by temperature signals. Of the isolations where there was actual leakage from the RWCU pressure boundary, the usual initiating isolation signal was related to flow not to area temperatures. Thus, a significant factor for RWCU pressure boundary leakage detection is flow monitoring. The licensee has not proposed any change in the flow monitoring instrumentation.

RCIC/RHR Steam Line High Flow Actuation - The licensee stated that the originally specified initial setpoint of $\leq 156''$ H₂O for flow measuring instrumentation to isolate the RCIC/RHR steam line on high flow is below the break flow for an 8" RHR steam line break but may not have been low enough to isolate on a 4" RCIC steam line break. This is due to errors in the original flow calculation which resulted in the overestimation of total steam flow. The errors resulted from the multiple operating modes of the RHR system. The proposed initial setpoint of $\leq 60.7''$ H₂O will isolate down stream piping on both the 4" and 8" steam line break and is based on steam flow to both loops of the RHR system in the steam condensing mode and to the RCIC taking suction from the condensate storage tank. Since the setpoint is being lowered in the conservative direction there will be an increase rather than a decrease in the margin of safety associated with this change. No change to the applicable note in the TS, Table 3.3.2-2 is proposed which states that the final setpoint is to be determined during testing prior to operation in the steam condensing mode following the NRC approval to operate in that mode.

3.0 SUMMARY

Based on the above, the staff concludes that the proposed change to the plant TS, Table 3.2.2-2 for leak detection setpoints for the RWCU heat exchanger room ambient high temperature, Item 4.c.1, and differential temperature, Item 4.d.1 are acceptable since the margin of safety as defined in the technical bases is maintained and all the equipment in the area is qualified for the environment to which it would be exposed.

The staff also concludes that the proposed changes to plant TS, Table 3.2.2-2, item 5.1, for leak detection initial setpoint for RCIC/RHR steam line high flow actuation is acceptable since the proposed change is more conservative than the original initial setpoint.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 5, 1988

Principal Contributors: R. Goel