

June 6, 1987

Docket No. 50-458

Mr. James C. Deddens
Senior Vice President, (RBNG)
Gulf States Utilities
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ATTN: Nuclear Licensing

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Dear Mr. Deddens:

SUBJECT: RIVER BEND STATION, UNIT 1, AMENDMENT NO. 6 TO FACILITY
OPERATING LICENSE NO. NPF-47 (TAC NO. 64830)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 6 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated January 28, 1987 as supplemented February 13 and April 16, 1987.

This amendment modifies the Scram Discharge Volume Water Level - High Trip set-point for the float switches LSN013A, B, C and D.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

IS)
Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 6
to License No. NPF-47
2. Safety Evaluation

cc w/enclosures:

See next page

PN
PD4/LA
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JCalvo
5/29/87

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P PDR

Mr. James C. Deddens
Gulf States Utilities Company

River Bend Nuclear Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by Gulf States Utilities Company, dated January 28, 1987 as supplemented February 13 and April 16, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 6 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Jose A. Calvo

Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 6, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. Overleaf page provided to maintain document completeness.*

REMOVE

2-3

2-4

INSERT

2-3*

2-4

SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.2 LIMITING SAFETY SYSTEM SETTINGS

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

2.2.1 The reactor protection system instrumentation setpoints shall be set consistent with the Trip Setpoint values shown in Table 2.2.1-1.

APPLICABILITY: As shown in Table 3.3.1-1.

ACTION:

With a reactor protection system instrumentation setpoint less conservative than the value shown in the Allowable Values column of Table 2.2.1-1, declare the channel inoperable and apply the applicable ACTION statement requirement of Specification 3.3.1 until the channel is restored to OPERABLE status with its setpoint adjusted consistent with the Trip Setpoint value.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE NO. NPF-47
GULF STATES UTILITIES COMPANY
RIVER BEND STATION, UNIT 1
DOCKET NO. 50-458

1.0 INTRODUCTION

By letter dated January 28, 1987 as supplemented February 13 and April 16 1987, Gulf States Utilities Company (GSU) (the licensee) requested an amendment to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. Technical Specification 2.2, Table 2.2.1-1, Reactor Protection System Instrumentation Setpoints, Item 9.b identifies the Scram Discharge Volume Water Level - High trip setpoint and allowable value for the Float Switches LSN013A, B, C and D. This proposed amendment requests a modification to the setpoints for this function.

These setpoints, which actuate a reactor trip, are provided to detect water level in the Scram Discharge Volume Water Level - High trip setpoint and allowable value for the Float Switches LSN013A, B, C and D. This proposed amendment requests a modification to the setpoints for this function.

These setpoints, which actuate a reactor trip, are provided to detect water level in the Scram Discharge Volume at a point high enough to indicate an accumulation of water but low enough to ensure that sufficient volume will remain to accept the water displaced from the movement of control rods when they are tripped.

The licensee contends that present design of the float switches has resulted in a configuration which cannot be calibrated to the Technical Specification setpoints but can be maintained below allowable values. During the review of the design, the licensee contends that sufficient margin is included in the original design to allow a revision of the trip setpoint although no change to the allowable value will be required. Therefore, the licensee is requesting the trip setpoints identified above be revised to less than or equal to 48.76 inches for LSN013A and B and less than or equal to 46.88 inches for LSN013C and D.

2.0 EVALUATION:

The initiation of the Reactor Protection System Trip is based on maintaining sufficient capacity in the scram discharge volume to accommodate the water displaced by the CRD pistons during a scram. The design function of the float switches has not changed and the minimum volume requirements have been maintained.

The proposed amendment changes the trip setpoint for float switches LSN013A, B, C and D, but the allowable values for these float switches remain unchanged. The Technical Specification allowable value are those values which are determined using acceptable methodology to ensure adequate conservatism exists to prevent exceeding safety limits during normal operation and design basis operational occurrences and to assist in mitigating the consequences of accidents.

The set point methodology used at River Bend assures that the process safety limit will not be exceeded. The licensee states and the staff agrees that this assurance is provided by the specific accounting of all inaccuracies between the process safety limit and the allowable value and by properly accounting for drift and calibration between the allowable value and the trip setpoint. There are additional margins due to conservative assumptions and calculations between the process safety limit and the trip set point which are not considered in the safety analysis. The requested change in trip set point modifies only this additional margin.

The margin between the allowable value and the top setpoint for LSN013A and B will be reduced from 6.18 to 4.74 inches. The margin for LSN013C and D will be reduced from 3.56 to 2.12 inches. The design of the float switches is such that there is a zero drift in switch actuation. The calibration of the float switches is such that the accuracy of calibrating them is estimated to be approximately + 0.02 inches. Since the margin that exists between the allowable value and the trip setpoint will always be greater than the maximum inaccuracy value of one inch, the new set-points assures an adequate margin of safety.

The staff also concurs with the licensees' assessment that the proposed change does not involve a significant reduction in the margin of safety and does not alter the River Bend Setpoint Methodology.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for

amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and the security nor to the health and safety of the public.

Principal Contributor: R. Lasky

Dated: June 6, 1987