April 21, 1988

Docket No. 50-458

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: CORRECTION TO AMENDENT NO. 19 TO FACILITY OPERATING LICENSE NO. NPF-47

On April 5, 1988, the Commission issued Amendment No. 19 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment changed the Technical Specifications (TS) for leak detection setpoints for the reactor water cleanup area temperatures and the RCIC/RHR steam line high flow isolation.

TS page 3/4 3-20 contained an error in which changed values in Amendment No. 1 were inadvertently omitted when the page was reissued in Amendment No. 19. The corrected TS page is enclosed. (TS page 3/4 3-19 is provided as an overleaf page to maintain document completeness.)

TS page 3/4 3-21 contained an error that was made when Amendment No. 11 was issued on September 23, 1987. The notation for a footnote (**) should have been deleted under item 5.a. (RCIC Steam Line Flow - High). When the page was reissued with Amendment No. 19 the error remained. The corrected TS page is enclosed.

TS page 3/4 3-22 contained an error in which a notation for a footnote (**) should have been added for item 5.1. (RHR/RCIC Steam Line Flow - High) but was inadvertently left off. The corrected TS page is enclosed.

Please accept our apology for any inconvenience these errors may have caused you.

8804290172 880421
PDR ADOCK 05000458
P PDR

Sincerely,
/s/
Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As stated
cc w/enclosures:
See next page

DISTRIBUTION

Docket File	NRC PDR	Local PDR	PD4 Reading
L. Rubenstein	P. Noonan	W. Paulson	OGC-Rockville
D. Hagan	E. Jordan	J. Partlow	T. Barnhart (4)
Wanda Jones	E. Butcher	ACRS (10)	GPA/PA
ARM/LFMB	PD4 Plant File	J. Calvo	

PD4/LA	PD4	PD4/D
PNoonan	WPaulson	JCalvo
04/21/88	04/21/88	04/21/88

April 21, 1988

Docket No. 50-458

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: CORRECTION TO AMENDENT NO. 19 TO FACILITY OPERATING LICENSE NO. NPF-47

On April 5, 1988, the Commission issued Amendment No. 19 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment changed the Technical Specifications (TS) for leak detection setpoints for the reactor water cleanup area temperatures and the RCIC/RHR steam line high flow isolation.

TS page 3/4 3-20 contained an error in which changed values in Amendment No. 1 were inadvertently omitted when the page was reissued in Amendment No. 19. The corrected TS page is enclosed. (TS page 3/4 3-19 is provided as an overleaf page to maintain document completeness.)

TS page 3/4 3-21 contained an error that was made when Amendment No. 11 was issued on September 23, 1987. The notation for a footnote (**) should have been deleted under item 5.a. (RCIC Steam Line Flow - High). When the page was reissued with Amendment No. 19 the error remained. The corrected TS page is enclosed.

TS page 3/4 3-22 contained an error in which a notation for a footnote (**) should have been added for item 5.1. (RHR/RCIC Steam Line Flow - High) but was inadvertently left off. The corrected TS page is enclosed.

Please accept our apology for any inconvenience these errors may have caused you.

Sincerely,
/s/
Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As stated
cc w/enclosures:
See next page

DISTRIBUTION

Docket File	NRC PDR	Local PDR	PD4 Reading
L. Rubenstein	P. Noonan	W. Paulson	OGC-Rockville
D. Hagan	E. Jordan	J. Partlow	T. Barnhart (4)
Wanda Jones	E. Butcher	ACRS (10)	GPA/PA
ARM/LFMB	PD4 Plant File	J. Calvo	
PD4/LA	PD4/PM	PD4/D	
PNoonan	WPaulson	JCalvo	
04/21/88	04/21/88	04/21/88	

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 21, 1988

Docket No. 50-458

Gulf States Utilities
ATTN: Mr. James C. Deddens
Senior Vice President (RBNG)
Post Office Box 220
St. Francisville, LA 70775

Dear Mr. Deddens:

SUBJECT: CORRECTION TO AMENDENT NO. 19 TO FACILITY OPERATING LICENSE NO. NPF-47

On April 5, 1988, the Commission issued Amendment No. 19 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment changed the Technical Specifications (TS) for leak detection setpoints for the reactor water cleanup area temperatures and the RCIC/RHR steam line high flow isolation.

TS page 3/4 3-20 contained an error in which changed values in Amendment No. 1 were inadvertently omitted when the page was reissued in Amendment No. 19. The corrected TS page is enclosed. (TS page 3/4 3-19 is provided as an overleaf page to maintain document completeness.)

TS page 3/4 3-21 contained an error that was made when Amendment No. 11 was issued on September 23, 1987. The notation for a footnote (**) should have been deleted under item 5.a. (RCIC Steam Line Flow - High). When the page was reissued with Amendment No. 19 the error remained. The corrected TS page is enclosed.

TS page 3/4 3-22 contained an error in which a notation for a footnote (**) should have been added for item 5.1. (RHR/RCIC Steam Line Flow - High) but was inadvertently left off. The corrected TS page is enclosed.

Please accept our apology for any inconvenience these errors may have caused you.

Sincerely,

Walter A. Paulson

Walter A. Paulson, Project Manager
Project Directorate - IV
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
See next page

Mr. James C. Deddens
Gulf States Utilities Company

River Bend Nuclear Plant

cc:
Troy B. Conner, Jr., Esq.
Conner and Wetterhahn
1747 Pennsylvania Avenue, NW
Washington, D.C. 20006

Mr. Les England
Director - Nuclear Licensing
Gulf States Utilities Company
P. O. Box 220
St. Francisville, LA 70775

Richard M. Troy, Jr., Esq.
Assistant Attorney General in Charge
State of Louisiana Department of Justice
234 Loyola Avenue
New Orleans, Louisiana 70112

Resident Inspector
P. O. Box 1051
St. Francisville, Louisiana 70775

Gretchen R. Rothschild-Reinike
Louisianians for Safe Energy, Inc.
2108 Broadway Street
New Orleans, Louisiana 70118-5462

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Office of Executive Director
for Operations
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Philip G. Harris
Cajun Electric Power Coop. Inc.
10719 Airline Highway
P. O. Box 15540
Baton Rouge, LA 70895

Mr. J. E. Booker
Manager-River Bend Oversight
P. O. Box 2951
Beaumont, TX 77704

Mr. William H. Spell, Administrator
Nuclear Energy Division
Office of Environmental Affairs
P. O. Box 14690
Baton Rouge, Louisiana 70898

Mr. J. David McNeill, III
William G. Davis, Esq.
Department of Justice
Attorney General's Office
7434 Perkins Road
Baton Rouge, Louisiana 70808

H. Anne Plettinger
3456 Villa Rose Drive
Baton Rouge, Louisiana 70806

President of West Feliciana
Police Jury
P. O. Box 1921
St. Francisville, Louisiana 70775

Mr. Frank J. Uddo
Uddo & Porter
6305 Elysian Fields Avenue
Suite 400
New Orleans, Louisiana 70122

TABLE 3.3.2-2
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level - Low Low, Level 2	≥-43 inches*	≥-47 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Containment Purge Isolation Radiation - High	≤ 1.3 R/hr	≤ 1.57 R/hr
2. MAIN STEAM LINE ISOLATION		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥-143 inches*	≥-147 inches
b. Main Steam Line Radiation - High	≤ 3.0 x full power background	≤ 3.6 x full power background
c. Main Steam Line Pressure - Low	≥ 849 psig	≥ 837 psig
d. Main Steam Line Flow - High		
1. Line A	≤ 146 psid	≤ 151 psid
2. Line B	≤ 156 psid	≤ 161 psid
3. Line C	≤ 153 psid	≤ 158 psid
4. Line D	≤ 164 psid	≤ 169 psid
e. Condenser Vacuum - Low	≥ 8.5 inches Hg. vacuum	≥ 7.6 inches Hg. vacuum
f. Main Steam Line Tunnel Temperature - High	≤ 135°F	≤ 142.5°F
g. Main Steam Line Tunnel Δ Temperature - High	≤ 51°F	≤ 55°F

8804290181 880421
PDR ADOCK 05000458
P PDR

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
2. MAIN STEAM LINE ISOLATION (Cont'd)		
h. Main Steam Line Area Temperature - High (Turbine Building)		
1. Main Steam Tunnel Area (El. 95')	≤ 142°F	≤ 145.3°F
2. Main Steam Tunnel Area (El. 114')	≤ 142°F	≤ 145.3°F
3. Main Steam Line Turbine Shield Wall	≤ 108°F	≤ 111.3°F
4. MSL Moisture Separator and Reheater Area	≤ 126°F	≤ 130°F
3. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level - Low Low Level 2	≥ - 43 inches*	≥ - 47 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Fuel Building Ventilation Exhaust Radiation - High		
1RMS*RE5A	≤ 1.82 x 10^3 μCi/sec	≤ 2.18 x 10^3 μCi/sec
1RMS*RE5B	≤ 5.88 x 10^{-4} μCi/cc	≤ 7.05 x 10^{-4} μCi/cc
d. Reactor Building Annulus Ventilation Exhaust Radiation - High	≤ 4.32 x 10^{-5} μCi/cc	≤ 5.19 x 10^{-5} μCi/cc
4. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High	≤ 55 gpm	≤ 62.1 gpm
b. Δ Flow Timer	≤ 45 seconds	≤ 47 seconds
c. Equipment Area Temperature - High		
1. Heat Exchanger Room	≤ 104.5°F	≤ 107.5°F
2. Pump Rooms A & B	≤ 165°F	≤ 169.5°F
3. Valve Nest Room	≤ 110°F	≤ 114.5°F
4. Demineralizer Rooms 1 and 2	≤ 110°F	≤ 114.5°F
5. Receiving Tank Room	≤ 110°F	≤ 114.5°F

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
4. REACTOR WATER CLEANUP SYSTEM ISOLATION (Cont'd)		
d. Equipment Area Δ Temperature - High		
1. Heat Exchanger Room	≤ 39°F	≤ 42.5°F
2. Pump Rooms A and B	≤ 78°F	≤ 82°F
3. Valve Nest Room	≤ 46°F	≤ 49.5°F
4. Demineralizer Rooms 1 and 2	≤ 46°F	≤ 49.5°F
5. Receiving Tank Room	≤ 46°F	≤ 49.5°F
e. Reactor Vessel Water Level - Low Low Level 2	≥ - 43 inches*	≥ - 47 inches
f. Main Steam Line Tunnel Ambient Temperature - High	≤ 135°F	≤ 142.5°F
g. Main Steam Line Tunnel Δ Temperature - High	≤ 51°F	≤ 55°F
h. SLCS Initiation	NA	NA
5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION		
a. RCIC Steam Line Flow - High	≤ 127" H_2O	≤ 135.5" H_2O
b. RCIC Steam Line Flow - High Timer	≥ 3 seconds	≤ 13 seconds
c. RCIC Steam Supply Pressure - Low	≥ 60 psig	≥ 55 psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10 psig	≤ 20 psig
e. RCIC Equipment Room Ambient Temperature - High	≤ 182°F	≤ 186.4°F
f. RCIC Equipment Room Δ Temperature - High	≤ 96°F	≤ 99°F

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION (Cont'd)		
g. Main Steam Line Tunnel Ambient Temperature - High	≤ 135°F	≤ 142.5°F
h. Main Steam Line Tunnel Δ Temperature - High	≤ 51°F	≤ 55°F
i. Main Steam Line Tunnel Temperature Timer	0 seconds	NA
j. RHR Equipment Room Ambient Temperature - High	≤ 117°F	≤ 121.1°F
k. RHR Equipment Room Δ Temperature - High	≤ 29°F	≤ 33.6°F
l. RHR/RCIC Steam Line Flow - High	≤ 60.7" H_2O**	≤ 64.2" H_2O**
m. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
n. Manual Initiation	NA	NA
6. RHR SYSTEM ISOLATION		
a. RHR Equipment Area Ambient Temperature - High	≤ 117°F	≤ 121.1°F
b. RHR Equipment Area Δ Temperature - High	≤ 29°F	≤ 33.6°F
c. Reactor Vessel Water Level - Low Level 3	≥ 9.7 inches*	≥ 8.7 inches
d. Reactor Vessel Water Level - Low Low Low Level 1	≥ - 143 inches*	≥ - 147 inches