

Docket No.: 50-458

NOV 20 1985

Mr. William J. Cahill, Jr.
Senior Vice President
River Bend Nuclear Group
Gulf States Utilities Company
Post Office Box 2951
Beaumont, Texas 77704
ATTN: Mr. J. E. Booker

Dear Mr. Cahill:

SUBJECT: ISSUANCE OF FACILITY OPERATING LICENSE NPF-47 RIVER BEND STATION,
UNIT 1

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Facility Operating License NPF-47, together with Technical Specifications and Environmental Protection Plan for the River Bend Station, Unit 1. License No. NPF-47 authorizes operation of the River Bend Station, Unit 1 at reactor power levels not in excess of 2894 megawatts thermal (100% rated power).

Enclosed is a copy of a related notice, the original of which has been forwarded to the Office of the Federal Register for publication.

Three signed copies of Amendment No. 2 to Indemnity Agreement No. B-104 which covers the activities authorized under License No. NPF-47 are also enclosed. Please sign all copies and return one copy to this office.

Sincerely,

Original signed by *

Hugh L. Thompson, Jr., Director
Division of Licensing
Office of Nuclear Reactor Regulation

Enclosures:

1. Facility Operating License NPF-47
2. Federal Register Notice
3. Amendment No. 2 to Indemnity Agreement No. B-104

cc: w/enclosures:
See next page

DL/LB#2
SStern ml
11/14/85

DL/LB#2
EHylton
11/ /85

SAB
ATEariston
11/14/85

SE
T...
11/15/85

OELD
JRutberg
11/18/85

OELD
L. Dewey
11/18/85

DL/LB#2
WButler
11/14/85

AD/L/DL
TNovak
11/15/85

DD/NRR
FMiraglia
11/19/85

D/DL
HThompson
11/19/85

DD/NRR
DEisenhut
11/ /85

D/NRR
HDenton
11/ /85

8511260559 851120
PDR ADOCK 05000458
P PDR



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NOV 20 1985

Docket No.: 50-458

Mr. William J. Cahill, Jr.
Senior Vice President
River Bend Nuclear Group
Gulf States Utilities Company
Post Office Box 2951
Beaumont, Texas 77704
ATTN: Mr. J. E. Booker

Dear Mr. Cahill:

SUBJECT: ISSUANCE OF FACILITY OPERATING LICENSE NPF-47 RIVER BEND STATION,
UNIT 1

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Facility Operating License NPF-47, together with Technical Specifications and Environmental Protection Plan for the River Bend Station, Unit 1. License No. NPF-47 authorizes operation of the River Bend Station, Unit 1 at reactor power levels not in excess of 2894 megawatts thermal (100% rated power).

Enclosed is a copy of a related notice, the original of which has been forwarded to the Office of the Federal Register for publication.

Three signed copies of Amendment No. 2 to Indemnity Agreement No. B-104 which covers the activities authorized under License No. NPF-47 are also enclosed. Please sign all copies and return one copy to this office.

Sincerely,

A handwritten signature in cursive script, reading "Hugh L. Thompson, Jr.", written over the typed name.

Hugh L. Thompson, Jr., Director
Division of Licensing
Office of Nuclear Reactor Regulation

Enclosures:

1. Facility Operating License NPF-47
2. Federal Register Notice
3. Amendment No. 2 to Indemnity Agreement No. B-104

cc: w/enclosures:
See next page

DISTRIBUTION

Docket File
NRC PDR
Local PDR
PRC System
NSIC
LB#2 Reading
EHylton*
BSiegel*
THovak
JSaltzman, SAB
Dewey, OELD*
CMiles
HDenton
JRutberg
AToalston
RDiggs, LFMB
JPartlow*
BGrimes*
EJordan*
LHarmon*
EButcher, SSPE*
TBarnhart (4)*
Inez Bailey
SStern*

Mr. William J. Cahill, Jr.
Gulf States Utilities Company

River Bend Nuclear Plant

cc:

Troy B. Conner, Jr., Esq.
Conner and Wetterhahn
1747 Pennsylvania Avenue, NW
Washington, D.C. 20006

Ms. Linda B. Watkins/Mr. Steven Irving
Attorney at Law
355 Napoleon Street
Baton Rouge, Louisiana 70802

Mr. William J. Reed, Jr.
Director - Nuclear Licensing
Gulf States Utilities Company
P. O. Box 2951
Beaumont, Texas 77704

Mr. David Zaloudek
Nuclear Energy Division
Louisiana Department of
Environmental Quality
P. O. Box 14690
Baton Rouge, Louisiana 70898

Richard M. Troy, Jr., Esq.
Assistant Attorney General in Charge
State of Louisiana Department of Justice
234 Loyola Avenue
New Orleans, Louisiana 70112

Mr. J. David McNeill, III
William G. Davis, Esq.
Department of Justice
Attorney General's Office
7434 Perkins Road
Baton Rouge, Louisiana 70808

Resident Inspector
P. O. Box 1051
St. Francisville, Louisiana 70775

H. Anne Plettinger
3456 Villa Rose Drive
Baton Rouge, Louisiana 70806

Gretchen R. Rothschild
Louisianians for Safe Energy, Inc.
1659 Glenmore Avenue
Baton Rouge, Louisiana 70775

James W. Pierce, Jr., Esq.
P. O. Box 23571
Baton Rouge, Louisiana 70893

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Office of Executive Director
for Operations
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

River Bend

cc: Attorney General
Department of Justice
State Capitol
Baton Rouge, Louisiana 79804

Office of Environmental Affairs
ATTN: Administrator, Nuclear
Energy Division
Post Office Box 14690
Baton Rouge, Louisiana 70898

President
West Feliciana Police Jury
Post Office Drawer N
St. Francisville, Louisiana 70775

Mr. Bruce Blanchard
Environmental Projects Review
U.S. Department of the Interior
18th & C Streets, N.W. - Room 4256
Washington, D.C. 20240

Defense Mapping Agency Aerospace
Center (ADL)
St. Louis Air Force Station, Missouri 63118



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY AND
CAJUN ELECTRIC POWER COOPERATIVE
DOCKET NO 50-458
RIVER BEND STATION, UNIT 1
FACILITY OPERATING LICENSE

License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for license filed by Gulf States Utilities Company, acting on behalf of itself and Cajun Electric Power Cooperative, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the River Bend Station, Unit 1 (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-145 and the application, as amended, the provisions of the Act and the regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - E. Gulf States Utilities Company* is technically qualified to engage in the activities authorized by this operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
 - F. Gulf States Utilities Company and Cajun Electric Power Cooperative have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;

*Gulf States Utilities Company is authorized to act as agent for Cajun Electric Power Cooperative and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Facility Operating License No. NPF-47, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Based on the foregoing findings and approval by the Nuclear Regulatory Commission at a meeting on November 15, 1985, the License for Fuel Loading and Low Power Testing, License No. NPF-40, issued on August 29, 1985, is superseded by Facility Operating License NPF-47 hereby issued to Gulf States Utilities Company and Cajun Electric Power Cooperative (the licensees), to read as follows:
- A. This license applies to the River Bend Station, Unit 1, a boiling water nuclear reactor and associated equipment, owned by Gulf States Utilities Company and Cajun Electric Power Cooperative. The facility is located approximately 2 miles east of the Mississippi River in West Feliciana Parish, Louisiana, approximately 2.7 miles southeast of St. Francisville, Louisiana and approximately 18 miles northwest of the city limits of Baton Rouge, Louisiana, and is described in the licensees' "Final Safety Analysis Report," as supplemented and amended, and in the licensees' Environmental Report-Operating License Stage, as supplemented and amended.
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
 - (1) Gulf States Utilities Company (GSU) and Cajun Electric Power Cooperative to possess the facility at the designated location in West Feliciana Parish, Louisiana, in accordance with the procedures and limitations set forth in this license;
 - (2) GSU, pursuant to Section 103 of the Act and 10 CFR Part 50, to use and operate the facility at the above designated location in accordance with the procedures and limitations set forth in this license;
 - (3) GSU, pursuant to the Act and 10 CFR Part 70, to receive, possess and to use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

- (4) GSU, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) GSU, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) GSU, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

GSU is authorized to operate the facility at reactor core power levels not in excess of 2894 megawatts thermal (100% rated power) in accordance with the conditions specified herein. The items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
 - (3) Antitrust Conditions

GSU shall comply with the antitrust conditions in Appendix C attached hereto, which is hereby incorporated into this license.

(4) Seismic and Dynamic Qualification of Seismic Category 1 Mechanical and Electrical Equipment (Section 3.10, SER and SSER 3)

GSU shall complete the requirements of the seismic and dynamic qualification of mechanical and electrical equipment as specified in Attachment 2. Attachment 2 is hereby incorporated into this license.

(5) Mark III Related Issues (Section 6.2.1.9, SER and SSER 2)

- a. GSU shall not use the residual heat removal system in the steam condensing mode without prior written approval of the staff.
- b. Prior to startup following the first refueling outage, GSU shall furnish the outstanding information identified in Appendix K of SSER 2 addressing the Mark III containment related issues.

(6) Inservice Inspection Program (Section 5.2.4.3 and 6.6.3, SER and SSER 3)

GSU shall submit the inservice inspection program for NRC staff review and approval by September 1, 1986.

(7) Bypassed and Inoperable Status Indication (Section 7.5.2.2, SER and SSER 3)

Prior to startup following the first refueling outage, GSU shall implement design modifications to improve the capabilities of existing bypassed and inoperable status indication used to monitor the status of safety related systems. The specific design changes to be implemented are identified in a GSU letter dated December 3, 1984 as clarified in a GSU letter dated March 5, 1985.

(8) TDI Diesel Engines (Section 8.3.1 SSER 3)

GSU shall implement the TDI diesel requirements as specified in Attachment 3. Attachment 3 is hereby incorporated into this license.

(9) Ultimate Heat Sink (Section 9.2.5, SER and SSER 3)

Prior to startup following the first refueling outage GSU shall have installed and operational in the ultimate heat sink a permanent temperature monitoring system acceptable to the NRC staff and Technical Specification modifications as required.

(10) Fire Protection (Section 9.5.1, SER and SSER 3)

GSU shall comply with the requirements of the fire protection program as specified in Attachment 4. Attachment 4 is hereby incorporated into this license.

(11) Operating Staff Experience Requirements (Section 13.1.2.1, SSER 2)

GSU shall have a licensed senior operator on each shift, while in Operating Condition 1, 2 and 3, who has had at least six months of hot operating experience on a plant comparable to River Bend Station, including at least six weeks at power levels greater than 20% of full power, and who has had startup and shutdown experience.

(12) Post-Fuel-Loading Initial Test Program (Section 14, SER and SSER 3)

Any changes to the initial test program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(13) Partial Feedwater Heating (Section 15.1, SER)

The facility shall not be operated with partial feedwater heating for the purpose of extending the normal fuel cycle without prior written approval of the staff.

(14) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737, Section 7.5.2.4, SER and SSER 3, Section 18, SER, SSER 2 and SSER 3)

GSU shall complete the requirements of NUREG-0737 Supplement #1 as specified in Attachment 5. Attachment 5 is hereby incorporated into this license.

(15) Salem ATWS Event, Generic Letter 83-28 (Section 7.2.2.5, SSER 3)

GSU shall submit responses to and implement the requirements of Generic Letter 83-28 on a schedule which is consistent with that given in its letters dated August 3, 1984 and May 20, 1985.

- D. GSU shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans, including all amendments and revisions made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). These plans, which contain safeguards information protected under 10 CFR 73.21, are entitled: "River Bend Station Physical Security Plan," "River Bend Station Security Training and Qualification Plan" and "River Bend Station Safeguards Contingency Plan."
- E. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, GSU shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).
- F. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- G. This license is effective as of the date of issuance and shall expire at midnight on August 29, 2025.

FOR THE NUCLEAR REGULATORY COMMISSION



Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Attachments 1-5
- 2. Appendix A - Technical Specifications (NUREG-1172)
- 3. Appendix B - Environmental Protection Plan
- 4. Appendix C - Antitrust Conditions

Date of Issuance: November 20, 1985

- D. GSU shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans, including all amendments and revisions made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). These plans, which contain safeguards information protected under 10 CFR 73.21, are entitled: "River Bend Station Physical Security Plan," "River Bend Station Security Training and Qualification Plan" and "River Bend Station Safeguards Contingency Plan."
- E. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, GSU shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).
- F. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- G. This license is effective as of the date of issuance and shall expire at midnight on August 29, 2025.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Attachments 1-5
- 2. Appendix A - Technical Specifications (NUREG-1172)
- 3. Appendix B - Environmental Protection Plan
- 4. Appendix C - Antitrust Conditions

Date of Issuance: **NOV 20 1985**

*Previously concurred:

DL/LB#2	DL/LB#2	SAB	SP	OELD	OELD
* SStern	EHylton	*ATOalston	*TSaltzman	*JRutberg	*LDewey
11/14 /85	11/ /85	11/14 /85	11/ 15 /85	11/15 /85	11/18 /85
DL/LB#2	AD/L/DL	DO/DA	D/DL	DD/NRR	D/NRR
*WButler	*TNovak	FMinaglia	HThompson	DEisenhut	HDeaton
11/ 14/85	11/ 15/85	11/ 14 /85	11/ 19 /85	11/ /85	11/ 20 /85

ATTACHMENT 1
TO-NPF 47

Prior to achieving the condition indicated on or before the date indicated, the following items shall be completed to the satisfaction of Region IV:

1. OUTSTANDING ITEMS TO BE ACCOMPLISHED PRIOR TO COMPLETION OF THE INITIAL TEST PROGRAM
 - a. Verify the station electric distribution voltage analyses are in accordance with the guidelines of Branch Technical Position PSB-1, Position 4.
 - b. Perform an engineering evaluation and complete modifications on the battery powered lighting system used in areas of the plant outside the main control room required for safe shutdown and personnel evacuation to upgrade those areas identified as deficient with regard to the requirements contained in FSAR Table 9.5-2
2. OUTSTANDING ITEM TO BE COMPLETED PRIOR TO THE FIRST REFUELING OUTAGE
 - a. Complete and have operational the fuel building sampling system. Off-loading of irradiated fuel prior to the first refueling outage shall be performed as described in GSU's letter dated June 13, 1985.
3. OUTSTANDING ITEM TO BE COMPLETED PRIOR TO FIRST DESIGN USE OR PRIOR TO STARTUP FOLLOWING THE FIRST REFUELING OUTAGE (WHICHEVER IS FIRST)
 - a. Complete a load test which meets all the OSHA requirements for load handling capability on all remaining cranes and hoists not previously tested.
4. OUTSTANDING ITEMS TO BE ACCOMPLISHED PRIOR TO STARTUP FOLLOWING THE FIRST REFUELING OUTAGE
 - a. Repair or replace the control valves on HVAC chillers as delineated in the 10 CFR 50.55(e) report identified as DR-314.
 - b. Verify that adequate radio communication capability exists from all appropriate plant areas.

ATTACHMENT 2

TO-NPF 47

SEISMIC AND DYNAMIC QUALIFICATION OF SEISMIC CATEGORY 1 MECHANICAL AND
ELECTRICAL EQUIPMENT

GSU shall complete the following requirements for seismic and dynamic qualification on the schedule noted below:

1. GSU shall, prior to startup following the second refueling outage, have completed modifications to the hydraulic control units to install an additional brace as used in the qualification testing of the hydraulic control unit as described in GSU's letter dated May 15, 1985.
2. GSU shall complete the seismic qualification of the in-vessel rack prior to its use.

ATTACHMENT 3
TO-NPF 47
TDI DIESEL ENGINES REQUIREMENTS

GSU shall comply with the following requirements related to the TDI diesel engines.

1. Changes to the maintenance and surveillance program for the TDI diesel engines, as identified and approved by the NRC staff in Supplement 3 to the SER, shall be subject to the provisions of 10 CFR 50.59.

2. Crankshafts shall be inspected as follows:

SD 1B: During the first refueling outage, inspect the fillets and oil holes of the three most heavily loaded crankpin journals (Nos. 5, 6, and 7) with fluorescent liquid penetrant and ET as appropriate.

SD 1A and 1B: During the second and subsequent refueling outages, inspect the fillets and oil holes of two of the three most heavily loaded crankpin journals in the manner just mentioned.

SD 1A and 1B: During each major engine overhaul, inspect the fillets and oil holes of the two main bearing journals between crankpin Nos. 5, 6, and 7, using fluorescent liquid penetrant and ET as appropriate. This inspection is in addition to the crankpin inspections.

3. Cylinder blocks shall be inspected at intervals calculated using the cumulative damage index (CDI) model and using inspection methodologies described by Failure Analysis Associates, Inc. in Design Review of TDI R-4 and RV-4 Series Emergency Diesel Generator Cylinder Blocks (FaAA-84-9-11-1). Liquid penetrant inspection of cylinder liner landing area shall be performed any time cylinder liners are removed. Visual daily inspection between adjacent cylinder heads and the general block top during any period of continuous operation following automatic diesel generator startup.
4. GSU shall roll the engines over with the air start system prior to any planned starts, unless that planned start occurs within four hours of a shutdown. In addition, after engine operation, the engines shall be rolled over on air after four hours but not more than eight hours after engine shutdown and then rolled over once again approximately 24 hours after each shutdown. In the event an engine is removed from service for any reason other than the rolling over procedure prior to expiration of the eight hour or 24 hour periods noted above, that engine need not be rolled over while it is out of service. Once the engine is returned to service, GSU shall roll it over with air once at the time that it is returned to service. Any head which leaks due to a crack shall be replaced.

5. If inspection of diesel generators 1A and/or 1B should reveal cracks in the crankshaft or in the cylinder block between stud holes of adjacent cylinders, this condition shall be reported promptly to the NRC staff and the affected engine(s) shall be considered inoperable. The engine(s) shall not be restored to "operable" status until the proposed disposition and/or corrective actions have been approved by the NRC staff.
6. The following actions are required if SD 1A or SD 1B is operated in excess of 3130 KW⁽¹⁾:
 - a) For indicated engine loads in the range of 3130 KW to 3200 KW for a period less than two hours⁽²⁾, no additional action shall be required.
 - b) For indicated engine loads in the range of 3130 KW to 3200 KW for a period equal to or exceeding two hours⁽²⁾, a crankshaft inspection pursuant to Item d below shall be performed at the next refueling outage.
 - c) For indicated engine loads in the range of 3200 KW to 3500 KW for a period less than 1 hour⁽²⁾, a crankshaft inspection pursuant to item d below shall be performed for the affected engine at the next refueling outage.
 - d) For indicated engine loads in the range of 3200 KW to 3500 KW for periods equal to or exceeding one hour⁽²⁾, and for engine loads exceeding 3500 KW for any period of time, (1) the engine shall be removed from service as soon as safely possible, (2) the engine shall be declared inoperable, and (3) the crankshaft shall be inspected. The crankshaft inspection shall include crankpin journal numbers 5, 6, and 7 (the most heavily loaded) and the two main journals in between using fluorescent liquid penetrant and eddy current as appropriate.
7. Operation beyond the first refueling outage is subject to NRC staff approval based on the staff's final review of the Owners Group generic findings and of the overall design review and quality revalidation program at River Bend.

(1) Momentary transients (not exceeding 5 seconds) due to changing of bus loads need not be considered as an overload.

(2) If there are multiple overload events within a given load range since the previous crankshaft inspection, then the time period criterion applies to the total accumulated time in that load range.

ATTACHMENT 4
TO-NPF 47
FIRE PROTECTION PROGRAM REQUIREMENTS

GSU shall comply with the following requirements of the fire protection program:

1. GSU shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility through Amendment 22 and as approved in the SER dated May 1984 and Supplement 3 dated August 1985 subject to provisions 2 and 3 below.
2. GSU may make no change to the approved fire protection program which would significantly decrease the level of fire protection in the plant without prior approval of the Commission. To make such a change GSU must submit an application for license amendment pursuant to 10 CFR 50.90.
3. GSU may make changes to features of the approved fire protection program which do not significantly decrease the level of fire protection without prior Commission approval provided (a) such changes do not otherwise involve a change in a license condition or technical specification or result in an unreviewed safety question (see 10 CFR 50.59), and (b) such changes do not result in failure to complete the fire protection program approved by the Commission prior to license issuance. GSU shall maintain, in an auditable form, a current record of all such changes, including an analysis of the effects of the change on the fire protection program, and shall make such records available to NRC inspectors upon request. All changes to the approved program shall be reported to the Director of the Office of Nuclear Reactor Regulation, along with the FSAR revisions required by 10 CFR 50.71(e).

ATTACHMENT 5
TO-NPF 47
EMERGENCY RESPONSE CAPABILITIES

GSU shall complete the following requirements of NUREG-0737 Supplement #1 on the schedule noted below:

1. Actions and schedules for correcting all human engineering discrepancies (HEDs) identified in the "Detailed Control Room Design Review Summary Report dated October 31, 1984 and Supplements dated May 14, June 12, 1985, and July 31, 1985, shall be implemented in accordance with the schedule committed to by GSU in the summary report and supplements and accepted by the NRC staff in Section 18.1 of SSER 3.

2. Prior to startup following the first refueling outage, GSU shall implement modifications (installation or upgrade) for those items listed below consistent with the guidance of Regulatory Guide 1.97, Revision 2 unless prior approval of an alternate design of these items is granted by the NRC staff. These items as listed in GSU's letter of June 24, 1985 are:
 - a) neutron flux;
 - b) coolant level in the reactor;
 - c) suppression pool water level;
 - d) drywell atmosphere temperature;
 - e) primary system safety relief valve position;
 - f) standby liquid control system storage tank level;
 - g) emergency ventilation damper position; and
 - h) airborne radiohalogens and particulates.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-458

RIVER BEND STATION, UNIT NO. 1

GULF STATES UTILITIES AND CAJUN ELECTRIC POWER COOPERATIVE

NOTICE OF ISSUANCE OF FACILITY

OPERATING LICENSE

Notice is hereby given that the Nuclear Regulatory Commission (the Commission), has issued Facility Operating License No. NPF-47 to the Gulf States Utilities and Cajun Electric Power Cooperative (licensees), which authorizes operation of the River Bend Station Unit No. 1 (the facility) at reactor core power levels not in excess of 2894 megawatts thermal in accordance with provisions of the License, the Technical Specifications and the Environmental Protection Plan.

The River Bend Station, Unit No. 1, is a boiling water nuclear reactor located approximately 2 miles east of the Mississippi River in West Feliciana Parish, Louisiana, approximately 2.7 miles southeast of St. Francisville, Louisiana and approximately 18 miles northwest of the city limits of Baton Rouge, Louisiana.

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the License. Prior public notice of the overall action involving the proposed issuance of an operating license was published in the Federal Register on September 4, 1981 (46 F.R. 44539-44540).

8511260568 851120
PDR ADOCK 05000458
P PDR

The Commission has determined that the issuance of this license will not result in any environmental impacts other than those evaluated in the Final Environmental Statement since the activity authorized by the license is encompassed by the overall action evaluated in the Final Environmental Statement.

For further details in respect to this action, see (1) Facility Operating License NPF-47 complete with Technical Specifications and the Environmental Protection Plan; (2) the interim report of the Advisory Committee on Reactor Safeguards, dated July 17, 1984; (3) the report of the ACRS dated September 16, 1985; (4) the Commission's Safety Evaluation Report, dated May 1984, Supplement No. 1 dated October 1984, Supplement No. 2 dated August 1985, Supplement No. 3 dated August 1985, and Supplement No. 4 dated September 1985; (5) the Final Safety Analysis Report and Amendments thereto; (6) the Environmental Report and supplements thereto; and (7) the Final Environmental Statement dated January 1985.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Government Documents Department, Louisiana State University, Baton Rouge, Louisiana 70803. A copy of Facility Operating License NPF-47 may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing. Copies of the Safety Evaluation Report and its Supplements 1, 2, 3, and 4 (NUREG-0989) and the Final Environmental Statement (NUREG-1073) may be purchased at current rates from the National Technical Information Service, Department of Commerce, 5285 Port Royal Road, Springfield, Virginia 22161, or may be ordered by calling (202) 275-2060

or (202) 275-2171 or by writing to the Superintendent of Documents, U.S. Government Printing Office, P.O. Box 37082, Washington, D.C. 20013-7082. All orders should clearly identify the NRC publication number and the requestor's GPO deposit account, or VISA or Mastercard number and expiration date.

Dated at Bethesda, Maryland this 20th day of November 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script that reads "Walter R. Butler".

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

or (202) 275-2171 or by writing to the Superintendent of Documents, U.S. Government Printing Office, P.O. Box 37082, Washington, D.C. 20013-7082. All orders should clearly identify the NRC publication number and the requestor's GPO deposit account, or VISA or Mastercard number and expiration date.

Dated at Bethesda, Maryland this 20th day of November 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

~~Original~~ signed by:

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

DL/LB#2
EHylton
11/ /85

DL/LB#2
SStern ml
11/14 /85

By Telephone
per S. Stern
OELD
C. Dewey
11/19 /85

DL/LB#2
WButler
11/14 /85



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-458

AMENDMENT TO INDEMNITY AGREEMENT NO. B-104
AMENDMENT NO. 2

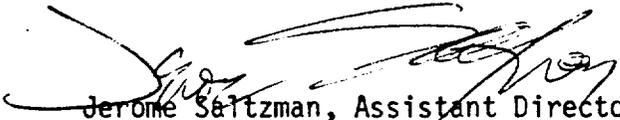
Effective November 20, 1985, Indemnity Agreement No. B-104, between Gulf States Utilities Company, Cajun Electric Power Cooperative, Inc., and the Nuclear Regulatory Commission, dated January 15, 1985, as amended, is hereby further amended as follows:

Item 3 of the Attachment to the indemnity agreement is deleted in its entirety and the following substituted therefor:

Item 3 - License number or numbers

SNM-1943	(From 12:01 a.m., January 15, 1985 to 12 midnight, August 28, 1985 inclusive)
NPF-40	(From 12:01 a.m., August 29, 1985 to 12 midnight, November 19, 1985 inclusive)
NPF-47	(From 12:01 a.m., November 20, 1985)

FOR THE U. S. NUCLEAR REGULATORY COMMISSION


Jerome Saltzman, Assistant Director
State and Licensee Relations
Office of State Programs

Accepted _____

Accepted _____

By _____
GULF STATES UTILITIES COMPANY

By _____
CAJUN ELECTRIC POWER
COOPERATIVE, INC.