July 10, 1995

Mr. George A. Hunger r.
Director-Licensing, Mc 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, PA 19087-0195

SUBJECT: ISSUANCE OF AMENDMENT-TYPE A CONTAINMENT INTEGRATED LEAKAGE RATE TEST INTERVAL, PEACH BOTTOM ATOMIC POWER STATION, UNIT 3 (TAC NO. M91028)

Dear Mr. Hunger:

The Commission has issued the enclosed Amendment No. 210 to Facility Operating License No. DPR-56 for the Peach Bottom Atomic Power Station, Unit 3. This amendment consists of changes to the Technical Specifications (TS) in response to your application dated November 21, 1994.

This amendment changes the TS by allowing the third Type A Containment Integrated Leakage Rate Test in the second 10-year service period to be conducted during refueling outage 11 scheduled for September 1997. This TS change is consistent with a one-time exemption from Appendix J to 10 CFR Part 50 that extends the 10-year service period and allows the three Type A tests to be performed at intervals that are not approximately equal.

You are requested to inform the staff, in writing, when this amendment has been implemented. This request affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. 96-511.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,
/S/
Joseph W. Shea, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-278

Enclosures: 1. Amendment No. 210 to License No. DPR-56
2. Safety Evaluation

cc w/enclosures: See next page

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REQ

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 10, 1995

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2. Safety Evaluation

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cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PECO ENERGY COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 210
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by PECO Energy Company, et al. (the licensee) dated November 21, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 210 , are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 10, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 210

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove	Insert
167	167

LIMITING CONDITIONS FOR OPERATION - SURVEILLANCE REQUIREMENTS

3.7.A Primary Containment (Cont'd.)

4.7.A Primary Containment (Cont'd.)

L_{tm} = measured ILR at 25 psig (P_t)

L_{am} = measured ILR at 49.1 psig (P_a), and

$\frac{L_{tm}}{L_{am}} \leq 0.7$, otherwise

$L_t = La\ (P_t/P_a)^{1/2}$

where

La = 0.5 percent of the primary containment volume per 24 hours at 49.1 psig

P_a = peak accident pressure (psig)

P_t = appropriately measured test pressures (psig)

c. The ILRT's shall be performed at the following minimum frequency:

1. Prior to initial unit operation.

2. After the preoperational leakage rate tests, a set of three Type A tests shall be performed at approximately equal intervals during each 10 year service period.* These intervals may be extended up to eight months if necessary to coincide with refueling outage.

d. The allowable leakage rates, L_{tm} and L_{am}, shall be less than 0.75 L_t and 0.75 L_a for the reduced pressure tests and peak pressure tests, respectively.

* Except for third Type A test in the second 10 year service period, which will be performed during the PBAPS, Unit 3 refueling outage 11 currently scheduled for September 1997.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 210 TO FACILITY OPERATING LICENSE NO. DPR-56

PECO ENERGY COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

DOCKET NO. 50-278

1.0 INTRODUCTION

By letter dated November 21, 1994, PECO Energy Company (PECO, the licensee) submitted a request for changes to the Peach Bottom Atomic Power Station (PBAPS), Unit No. 3, Technical Specifications (TS). The requested changes would allow an extension of the second 10-year Type A Containment Integrated Leak Rate Test (CILRT) service period and an extended interval between the second and third Type A tests in the second 10-year period.

2.0 EVALUATION

The existing TS Surveillance Requirement 4.7.A.2.c.2 states that "After the preoperational leakage rate tests, a set of three Type A tests shall be performed at approximately equal intervals during each 10-year service period." This one-time change would extend the Type A surveillance test service period, and increase the elapsed time since the last Type A test (December 1991). This extension will allow performing three CILRTs, instead of four CILRTs, within the second 10-year service period. The benefit of not performing the additional CILRT is a reduction in personnel radiation exposure. A dose saving will be realized from eliminating contamination, reducing exposure for venting and draining, and from setup and restoration of instrumentation required to perform the test. The TS change is implemented by the addition of a footnote to item 4.7.A.2.c.2 of the TS explaining the one-time extension.

The TS change could introduce the possibility that primary containment leakage in excess of the allowable value remain undetected during the proposed 24-month extension of the interval between the performance of the second and the third Type A test for the PBAPS Unit 3 primary containment. There are two types of mechanisms which could cause the degradation of the containment: (1) degradation due to a modification or maintenance activity on a component or system (i.e., activity-based), and (2) degradation resulting from a time-based failure mechanism. The licensee performed a review of the history of the PBAPS Unit 3 CILRT results to evaluate the risk of activity-based and

time-based degradation. This review identified three activity-based component failures detected during past CILRTs. The measured mass point and total time leakage rates measured for the April 1977 CILRT stabilized at approximately 1.1% wt/day, which failed to meet the TS and 10 CFR Part 50, Appendix J criterion of less than 0.375% wt/day (0.75 La). Following the completion of repairs of a leaking torus water level instrument, the CILRT was repeated with an as-left leakage of 0.322% wt/day. After this failure, the licensee modified the plant procedures so that a similar failure, in the future, would be detected by a local leak rate test (LLRT). The measured mass point and total time leakage rates measured for the September 1981 CILRT stabilized at approximately .389% wt/day, which failed to meet the TS and 10 CFR Part 50, Appendix J criterion of less than 0.375% wt/day (0.75 La). Following the completion of repairs to a missing instrument O-ring, the CILRT was repeated with an as-left leakage of 0.185% wt/day. After this failure, the licensee modified the plant procedures so that a similar failure, in the future, would be detected by a leak rate test following relevant instrument maintenance. The measured mass point and total time leakage rates measured for the August 1983 CILRT stabilized at approximately .784% wt/day, which failed to meet the TS and 10 CFR Part 50, Appendix J criterion of less than 0.375% wt/day (0.75 La). Following the completion of repairs to a valve packing leak, the CILRT was repeated with an as-left leakage of 0.058% wt/day. After this failure, the licensee modified the plant procedures so that similar valve packing is local leak rate tested and leakage is measured.

According to information provided by the licensee, during the second 10-year Type A service interval (commencing in July 1984), the licensee has not experienced any as-found failures (either activity-based or time-based) during Unit 3 Type A tests. Three satisfactory tests have been conducted in the second service period (January 1986, November 1989 and December 1991).

The Type B and C test (Local Leak Rate Test (LLRT)) program also provides assurance that containment integrity has been maintained. LLRTs demonstrate operability of components and penetrations by measuring penetration and valve leakage. Additionally, there have been no modifications made to the plant that could adversely affect the test results.

Current TS 4.7.A.2.h requires that the interior surfaces of the drywell and torus shall be visually inspected each operating cycle for evidence of deterioration. In addition, TS 4.7.A.2.h requires that the external surfaces of the torus below the water level be inspected on a routine basis for evidence of torus corrosion or leakage. TS 4.7.4 requires that a visual inspection of the suppression chamber interior be conducted at each major refueling outage. These inspections provide similar information as would be obtained to meet the requirement of Section V.A of 10 CFR Part 50, Appendix J. The licensee is required to perform these TS surveillances in the upcoming refueling outage (Unit 3 refueling outage 10).

Since the licensee has justified the leaktight integrity of the containment based on previous leakage test results, and because the licensee will perform

visual inspection of the primary containment during the upcoming refueling outage, the staff concludes that a one-time extension beyond the maximum permitted test interval will not have a significant safety impact. Therefore, the staff concludes that the licensee's request of a one-time extension of the 10-year service period and a one-time extension of the test interval between consecutive CILRTs for the second 10-year service period is acceptable.

The licensee proposed to implement the one-time TS change by adding a footnote to TS 4.7.A.2.c.2 detailing the extended interval.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 27340). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Shea

Date: July 10, 1995