

UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

February 25, 1992

Docket Nos. 50-277
and 50-278

Posted
Amnt. 170 to DPR-56

Mr. George J. Beck
Manager-Licensing, MC 5-2A-5
Philadelphia Electric Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

Dear Mr. Beck:

SUBJECT: TECHNICAL SPECIFICATION DEFINITION OF SURVEILLANCE FREQUENCIES FOR
PEACH BOTTOM, ATOMIC POWER STATION, UNITS 2 AND 3 (TAC NOS. M82322
AND M82323)

The Commission has issued the enclosed Amendments Nos. 166 and 170 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications in response to your application dated December 19, 1991.

These amendments revise the Technical Specification definition of Surveillance Frequency. The revised definition provides specific definitions of the surveillance intervals used throughout the Technical Specifications. In addition, the changes revise the reference date from which subsequent surveillance tests are scheduled when the actual interval exceeded the specified interval by up to 25% of that specified. Finally, the changes delete potentially conflicting words from the definition. It is understood that the amendments will be placed into effect upon successful implementation of the Plant Information Management System Surveillance Testing Module, currently scheduled for May, 1992. If program delays are encountered such that implementation will not be complete by May 31, 1992, notify the NRC staff in writing of your revised schedule prior to May 31, 1992. In addition, please notify the staff upon successful implementation of the Plant Information Management System Surveillance Testing Module.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

Joseph W. Shea, Acting Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 166 to DPR-44
- 2. Amendment No. 170 to DPR-56
- 3. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION w/enclosures:

Docket File CGrimes, 11E-21
 NRC & Local PDR ACRS(10)
 PDI-2 Rdg. GPA/PA
 SVarga OC/LFMB
 JCalvo RBlough, RGN-I
 CMiller LDoerflein, RGN-I
 MO'Brien(2)
 JShea
 OGC
 DHagan, 3206
 GHill(8), P1-37
 Wanda Jones, 7103

*Previously declined Concurrence
**Previously Concurred

OFC	:PDI-2/LA	:PDI-2/PM	:OTSB/EC*	:OGC**	:PDI-2/D
NAME	:MO'Brien	:JShea: <i>RS</i>	:CGrimes	:	:CMiller <i>J</i>
DATE	: 1/19/92	: 1/25/92	:02/19/92	:01/27/92	02/25/92

OFFICIAL RECORD COPY
Document Name: PB2/3 AMDT M82322/3

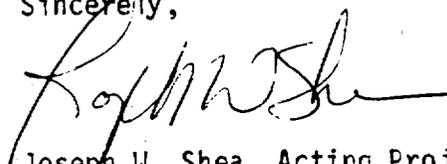
Mr. George J. Beck

- 2 -

February 25, 1992

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,



Joseph W. Shea, Acting Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 166 to DPR-44
2. Amendment No. 170 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. George J. Beck
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

J. W. Durham, Sr., Esquire
Sr. V.P. & General Counsel
Philadelphia Electric Company
2301 Market Street, S26-1
Philadelphia, Pennsylvania 19101

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Philadelphia Electric Company
ATTN: Mr. D. B. Miller, Vice President
Peach Bottom Atomic Power Station
Route 1, Box 208
Delta, Pennsylvania 17314

Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Philadelphia Electric Company
ATTN: Regulatory Engineer, A1-2S
Peach Bottom Atomic Power Station
Route 1, Box 208
Delta, Pennsylvania 17314

Public Service Commission of Maryland
Engineering Division
ATTN: Chief Engineer
231 E. Baltimore Street
Baltimore, MD 21202-3486

Resident Inspector
U.S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
P.O. Box 399
Delta, Pennsylvania 17314

Mr. Richard McLean
Power Plant and Environmental
Review Division
Department of Natural Resources
D-3, Tawes States Office Building
Annapolis, Maryland 21401

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Mr. George J. Beck
Manager-Licensing, MC 5-2A-5
Philadelphia Electric Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

Mr. Roland Fletcher
Department of Environment
201 West Preston Street
Baltimore, Maryland 21201

Carl D. Schaefer
External Operations - Nuclear
Delmarva Power & Light Company
P.O. Box 231
Wilmington, DE 19899



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 166
License No. DPR-44

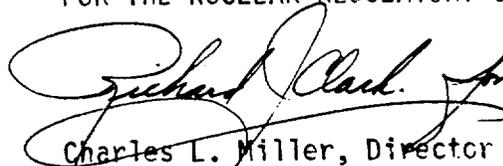
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et. al. (the licensee) dated December 19, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 166, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective upon implementation of the Plant Information Monitoring System Surveillance Testing Module.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 25, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 166

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove

8

Insert

8

1.0 DEFINITIONS (Cont'd)

Simulated Automatic Actuation - Simulated automatic actuation means applying a simulated signal to the sensor to actuate the circuit in question.

Site Boundary - That line beyond which the land is not owned, leased or otherwise controlled by licensee.

Source Check - A source check shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

Startup/Hot Standby Mode - In this mode the reactor protection scram trips, initiated by condenser low vacuum and main steam line isolation valve closure are bypassed, the reactor protection system is energized with IRM neutron monitoring system trip, the APRM 15% high flux trip, and control rod withdrawal interlocks in service. This is often referred to as just Startup Mode. This is intended to imply the Startup/Hot Standby position of the mode switch.

Surveillance Frequency - Periodic surveillance tests, checks, calibrations, and examinations shall be performed within the specified surveillance intervals. Specified periodic surveillance intervals are defined as:

(N) Hours	At least once per (N) hours
Shiftly	At least once per 12 hours
Daily	At least once per 24 hours
(N) Days	At least once per (N) days
Twice Per Week	At least once per 4 days
Weekly	At least once per 7 days
(N) Weeks	At least once per (7xN) days
Semi monthly	At least once per 15 days
Monthly	At least once per 31 days
2 Month	At least once per 61 days
Quarterly or 3 month	At least once per 92 days
Semi-annually or 6 month	At least once per 184 days
Annually or 12 month	At least once per 366 days
Once Per Cycle	At least once per 550 days
18 month	At least once per 550 days
Refuel	At least once per 550 days
(N) Years	At least once per (366xN) days
(N) Refuel Cycle	At least once per (550xN) days

These specified time intervals may be exceeded by 25%. Surveillance tests are not required on systems or parts of the systems that are not required to be operable or are tripped. If tests are missed on parts not required to be operable or are tripped, then they shall be performed prior to returning the system to an operable status.

A surveillance test of the diesel generators, that requires a plant outage, may be deferred beyond the calculated due date until the next refueling outage, provided the equipment has been similarly tested and meets the surveillance requirement for the other unit.

Transition Boiling - Transition boiling means the boiling regime between nucleate and film boiling. Transition boiling is the regime in which both nucleate and film boiling occur intermittently with neither type being completely stable.

Trip System - A trip system means an arrangement of instrument channel trip signals and auxiliary equipment required to initiate



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et. al. (the licensee) dated December 19, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 170, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective upon implementation of the Plant Information Monitoring System Surveillance Testing Module.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 25, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 170

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove

8

Insert

8

1.0 DEFINITIONS (Cont'd)

Simulated Automatic Actuation - Simulated automatic actuation means applying a simulated signal to the sensor to actuate the circuit in question.

Site Boundary - That line beyond which the land is not owned, leased or otherwise controlled by licensee.

Source Check - A source check shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

Startup/Hot Standby Mode - In this mode the reactor protection scram trips, initiated by condenser low vacuum and main steam line isolation valve closure are bypassed, the reactor protection system is energized with IRM neutron monitoring system trip, the APRM 15% high flux trip, and control rod withdrawal interlocks in service. This is often referred to as just Startup Mode. This is intended to imply the Startup/Hot Standby position of the mode switch.

Surveillance Frequency - Periodic surveillance tests, checks, calibrations, and examinations shall be performed within the specified surveillance intervals. Specified periodic surveillance intervals are defined as:

(N) Hours	At least once per (N) hours
Shiftly	At least once per 12 hours
Daily	At least once per 24 hours
(N) Days	At least once per (N) days
Twice Per Week	At least once per 4 days
Weekly	At least once per 7 days
(N) Weeks	At least once per (7xN) days
Semi monthly	At least once per 15 days
Monthly	At least once per 31 days
2 Month	At least once per 61 days
Quarterly or 3 month	At least once per 92 days
Semi-annually or 6 month	At least once per 184 days
Annually or 12 month	At least once per 366 days
Once Per Cycle	At least once per 550 days
18 month	At least once per 550 days
Refuel	At least once per 550 days
(N) Years	At least once per (366xN) days
(N) Refuel Cycle	At least once per (550xN) days

These specified time intervals may be exceeded by 25%. Surveillance tests are not required on systems or parts of the systems that are not required to be operable or are tripped. If tests are missed on parts not required to be operable or are tripped, then they shall be performed prior to returning the system to an operable status.

A surveillance test of the diesel generators, that requires a plant outage, may be deferred beyond the calculated due date until the next refueling outage, provided the equipment has been similarly tested and meets the surveillance requirement for the other unit.

Transition Boiling - Transition boiling means the boiling regime between nucleate and film boiling. Transition boiling is the regime in which both nucleate and film boiling occur intermittently with neither type being completely stable.

Trip System - A trip system means an arrangement of instrument channel trip signals and auxiliary equipment required to initiate



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENTS NOS. 166 AND 170 TO FACILITY OPERATING
LICENSE NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated December 19, 1991, the Philadelphia Electric Company (PECo), Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company (the licensees) submitted a request for changes to the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3, Technical Specifications (TS). The requested changes would revise the TS definition of Surveillance Frequency. Specifically, the proposed changes define each of the surveillance intervals used throughout the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3 TS in common units of hours and days. In addition, the proposed changes revise the reference date from which subsequent surveillance tests are scheduled when the actual interval exceeded the specified interval by up to 25% of that specified. Finally, the proposed changes remove a definition of the operating cycle that was stated in units of months since it is now defined in units of days. PECo proposed the changes to facilitate implementation of a new surveillance test scheduling and tracking system, referred to as the Plant Information Management System (PIMS) Surveillance Test Module. As documented in Inspection Reports 50-277/90-01, 50-278/90-01, 50-277/91-23 and 50-278/91-23, the licensee has had difficulty in completing some TS required surveillance tests within their required periodicity.

Discussion

In their submittal of December 19, 1991, the licensee proposed four specific changes to the Technical Specification 1.0 definition of Surveillance Frequency. The first change (Change Request (1)) adds a tabular list of the various surveillance intervals used throughout the TSs and define the intervals in units of hours and days. The licensee included the applicable surveillance intervals from Table 1.1 of NUREG-0123, Revision 3, "Standard Technical Specifications for General Electric Boiling Water Reactors." In addition, the licensee defined other intervals that are used in the Peach Bottom TSs but that are not listed in Table 1.1 of NUREG-0123. These were included for completeness and to avoid misinterpretation.

The second change (Change Request (2)) deletes a sentence defining the length of an "operating cycle" as an interval not to exceed 18 months. The table included in the first change defines an "operating cycle" as 550 days. This change eliminates conflicting requirements since a month is defined as 31 days and 18 months would constitute 558 days.

The third change (Change Request (3)) deletes a requirement in the definition of Surveillance Frequency that in cases where the elapsed interval has exceeded 100% of the specified interval, the next interval shall commence at the end of the original specified interval. This requirement results in scheduling completion of subsequent tests on the same periodic basis as would have been used if the surveillance had been performed within the specified interval. This has the effect of limiting the repeated use of the provision for extending surveillance intervals. Because the NRC has concluded in Generic Letter 89-14 that a similar requirement in the standard technical specifications is not necessary, the licensee has proposed Change Request (3). This change will allow the scheduling of each subsequent surveillance test based upon the date that the prior surveillance test was actually performed and this is consistent with the process used by the PIMS Surveillance Test Module.

The fourth change (Change Request (4)) deletes a requirement from the current definition which reads: "When a test is deferred under this provision, the next surveillance interval shall commence at the end of the original specified interval." This sentence is deleted from the paragraph in the definition that addresses testing of the common diesel generator related equipment. This deletion is proposed for the same reason as Change Request (3) and allows surveillance tests to be scheduled based on the date of the last completed test.

2.0 EVALUATION

Change Request (1) provides a table that specifically defines, in terms of hours and days, the surveillance intervals used throughout the Peach Bottom Technical Specifications. By providing a more precise definition of the intervals, the possibility of non-conservative interpretations is reduced. The majority of the intervals defined in the table are taken from Table 1.1 of NUREG-0123, Revision 3, "Standard Technical Specifications for General Electric Boiling Water Reactors" (STS). The remaining intervals in the table, which are used in various places throughout the Peach Bottom Technical Specifications, are included so as to eliminate confusion and to avoid misinterpretation. Because the proposed change is consistent with NUREG-0123, the staff finds the proposed change acceptable.

Change Request (2) deletes the portion of the current definition which reads: "The operating cycle interval as pertaining to instrument and electrical surveillance shall not exceed 18 months. Because the table in change request (1) provides a precise definition of the operating cycle interval, the staff finds the proposed change acceptable.

Change Request (3) deletes the requirement that subsequent surveillance tests be scheduled based on the last scheduled interval. Deletion of this requirement will allow tests to be scheduled based on the date of the last completed test. Generic Letter 89-14 concluded that there was safety benefit, in some instances, in invoking the twenty-five percent extension provision that the benefit to safety and would exceed any safety benefit derived from limiting use of the twenty-five percent allowance to extend a surveillance. On the basis that Generic Letter 89-14 supports the use of the twenty-five percent interval extension in circumstances where it benefits safe plant operation and allows scheduling of surveillance tests based on the date of the last completed test, the staff finds the proposed change acceptable.

Change Request (4) addresses the paragraph in the definition which addresses surveillance testing of the shared EDG's. The TSs allow the EDG testing required once per operating cycle to be deferred until the next refueling outage provided that the equipment had been similarly tested and met the surveillance requirement of the other unit. Change Request (4) deletes the requirement to schedule EDG testing based on previous schedules. This change is similar to that for Change Request (3) and on the basis of the evaluation presented for Change Request (3), the staff finds Change Request (4) acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATIONS

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (57 FR 714). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Shea, NRR

Dated: February 25, 1992