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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DO NOT REMOVE

February 18, 1983

Dockets Nos. 50-277  
and 50-278

Posted  
Amndt. 87  
to DPR-56

Mr. Edward G. Bauer, Jr.  
Vice President and General Counsel  
Philadelphia Electric Company  
2301 Market Street  
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

SUBJECT: TECHNICAL SPECIFICATION AMENDMENT PERTAINING TO SURVEILLANCE  
REQUIREMENTS TO PERMIT AN ALTERNATE METHOD OF PERFORMING  
CORE SPRAY LOOP FLOW RATE TEST

The Commission has issued the enclosed Amendments Nos. 87 and 87 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 1, 1982.

The changes to the TSs permit an alternate method of performing the core spray loop flow rate surveillance test.

Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Gerald E. Gears, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 87 to DPR-44
2. Amendment No. 87 to DPR-56
3. Safety Evaluation
4. Notice

cc w/enclosures:  
See next page

Philadelphia Electric Company

cc w/enclosure(s):

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Board of Supervisors  
Peach Bottom Township  
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Delta, Pennsylvania 17314

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87  
License No. DPR-44

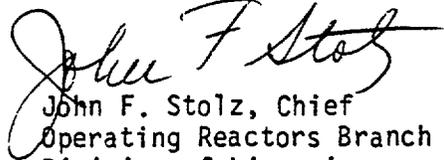
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated December 1, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
: Changes to the Technical  
: Specifications

Date of Issuance: February 18, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 87

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following page and add the new page of the Appendix "A" .  
Technical Specifications with the enclosed pages. The pages are  
identified by Amendment number and contain a vertical line indicating  
the area of change.

Remove

125

Insert

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125a

**LIMITING CONDITION  
FOR OPERATION**

**3.5.A Core Spray &  
LPCI Subsystem (cont'd)**

Both CSS shall be operable whenever irradiated fuel is in the vessel and prior to reactor startup from a Cold Shutdown condition except as specified in 3.5.A.2 and 3.5.F.3 below:

2. From and after the date that one of the core spray subsystems is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days provided that during such seven days all active components of the other core spray subsystem and active components of the LPCI subsystem are operable.

**SURVEILLANCE REQUIREMENTS**

**4.5.A Core Spray &  
LPCI Subsystem (cont'd)**

<u>Item</u>	<u>Frequency</u>
(d) Pump Flow Rate	Once/3 months
*Each Pump in each loop shall deliver at least 3125 gpm against a system head corresponding to a reactor vessel pressure of 105 psig.	
(e) Core Spray Header ΔP Instrumentation	
Check	Once/day
Calibrate	Once/3 months
(f) Operability	In accordance with 4.5.A.2, 4.5.A.4 and 4.5.A.5.
check to ensure that pumps will start and motor operated injection valves will open.	

2. When it is determined that one core spray subsystem is inoperable, the operable core spray subsystem and the LPCI subsystems shall be demonstrated to be operable in accordance with 4.5.A.1(f) and 4.5.A.3(e) within 24 hours and at least once per 72 hours thereafter until the inoperable core spray subsystem is restored to operable status.
3. LPCI Subsystem Testing shall be as follows:

\*Until the required modification is completed, the loop flow rate test at 6250 gpm against a system head corresponding to a reactor vessel pressure of 105 psig will be performed to satisfy surveillance requirements.

LIMITING CONDITIONS  
FOR OPERATION

## SURVEILLANCE REQUIREMENTS

<u>Item</u>	<u>Frequency</u>
(a) Simulated Automatic Actuation Test	Once/operating Cycle
(b) Pump operability	Once/1 month

-125a-



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87  
License No. DPR-56

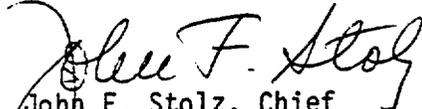
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated December 1, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 18, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 87

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following page and add the new page of the Appendix "A" .  
Technical Specifications with the enclosed pages. The pages are  
identified by Amendment number and contain a vertical line indicating  
the area of change.

Remove

125

Insert

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125a

**LIMITING CONDITIONS  
FOR OPERATION**

**3.5.A Core Spray &  
LPCI Subsystem (cont'd)**

Both CSS shall be operable whenever irradiated fuel is in the vessel and prior to reactor startup from a Cold Shutdown condition except as specified in 3.5.A.2 and 3.5.F.3 below:

2. From and after the date that one of the core spray subsystems is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days provided that during such seven days all active components of the other core spray subsystem and active components of the LPCI subsystem are operable.

**SURVEILLANCE REQUIREMENTS**

**4.5.A Core Spray &  
LPCI Subsystem (cont'd)**

<u>Item</u>	<u>Frequency</u>
(d) Pump Flow Rate	Once/3 months
*Each Pump in each loop shall deliver at least 3125 gpm against a system head corresponding to a reactor vessel pressure of 105 psig.	
(e) Core Spray Header ΔP Instrumentation	
Check	Once/day
Calibrate	Once/3 months
(f) Operability check to ensure that pumps will start and motor operated injection valves will open.	In accordance with 4.5.A.2, 4.5.A.4 and 4.5.A.5.

2. When it is determined that one core spray subsystem is inoperable, the operable core spray subsystem and the LPCI subsystems shall be demonstrated to be operable in accordance with 4.5.A.1(f) and 4.5.A.3(e) within 24 hours and at least once per 72 hours thereafter until the inoperable core spray subsystem is restored to operable status.
3. LPCI Subsystem Testing shall be as follows:

\*Until the required modification is completed, the loop flow rate test at 6250 gpm against a system head corresponding to a reactor vessel pressure of 105 psig will be performed to satisfy surveillance requirements.

LIMITING CONDITION  
FOR OPERATION

SURVEILLANCE REQUIREMENTS

<u>Item</u>	<u>Frequency</u>
(a) Simulated Automatic Actuation Test	Once/operating Cycle
(b) Pump operability	Once/1 month



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING  
AMENDMENTS NOS. 87 AND 87 TO FACILITY OPERATING LICENSES NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

INTRODUCTION

By letter dated December 1, 1982, the Philadelphia Electric Company (PECo or the licensee) made application to amend the Technical Specifications (TSs) for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3, to permit an alternate method of performing the core spray loop flow rate test. The licensee indicated that in order to eliminate minor cavitation and vibration in the core spray test valve, which occurs when performing the two pump loop flow test, single pump flow rate testing was being proposed.

EVALUATION

The licensee indicated in their December 1, 1982, submittal that modifications to the core spray loop flow rate surveillance testing were being proposed to eliminate cavitation and vibration in the core spray test valve which occurs when performing the two pump loop flow test as required by the TSs. Through resizing a restricting orifice (RO-42) and locating it further upstream of the core spray test valve (MO-26) and by adding another restricting orifice (RO-42-1) downstream of the valve, the licensee indicates that cavitations and vibrations will be eliminated. These modifications will require an alternate method of performing core spray loop flow rate testing; namely, single pump flow rate testing versus two pump loop flow rate testing.

As stated in the current Surveillance Requirements of TS Section 4.5.A.(d) for Units 2 and 3, the two pump loop flow test should deliver at least 6250 gpm against a system head corresponding to a reactor vessel pressure of 105 psig. The licensee's alternative method would provide that each pump in each loop would be tested to ensure the delivery of at least 3125 gpm against a system head corresponding to a reactor vessel pressure of 105 psig.

We have determined through a review of the licensee's submittal that proper sizing of the restricting orifices will provide verification of individual pump capacity. Furthermore, we have determined that verification of individual pump capacity demonstrates proper loop flow capability.

We conclude, based upon the above considerations, that the proposed changes to the TSS are acceptable.

#### ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 18, 1983

The following NRC personnel have contributed to this Safety Evaluation: Gerry Gears, Wayne Hodges, and Joel Page.

UNITED STATES NUCLEAR REGULATORY COMMISSION  
DOCKETS NOS. 50-277 AND 50-278  
PHILADELPHIA ELECTRIC COMPANY, ET AL  
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 87 and 87 to Facility Operating Licenses Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications (TSs) for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The revised TSs permit changes to the surveillance requirements allowing an alternate method of performing the core spray loop flow rate test.

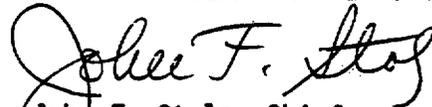
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendment dated December 1, 1982, (2) Amendment No. 87 to License No. DPR-44 and Amendment No. 87 to License No. DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 18th day of February 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing