

May 25, 1990

Docket No. 50-278

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Mr. George A. Hunger, Jr.
 Director-Licensing, MC 5-2A-5
 Philadelphia Electric Company
 Nuclear Group Headquarters
 Correspondence Control Desk
 P.O. Box No. 195
 Wayne, Pennsylvania 19087-0195

Dear Mr. Hunger:

SUBJECT: SNUBBER VISUAL INSPECTION FREQUENCY, PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3 (TAC NO. 76485)

The Commission has issued the enclosed Amendment No. 156 to Facility Operating License No. DPR-56 for the Peach Bottom Atomic Power Station, Unit No. 3. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 12, 1990, as supplemented on May 18, 1990. The supplemental letter provided clarifying information related to the licensee's safety evaluation in support of the requested license amendment. The staff has determined that the supplemental information does not affect the proposed no significant hazards determination.

This amendment allows a one time extension of about seven months for the performance of required visual inspections of inaccessible snubbers.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/S/

Gene Y. Suh, Project Manager
 Project Directorate I-2
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 156 to License No. DPR-56
2. Safety Evaluation

cc w/enclosures:
 See next page

[76485]

PDI-2/LA
 MO'Brien
 5/21/90

PDI-2/PM
 GSuh:mj
 5/21/90

OGC
 L. Dew
 5/23/90

PDI-2/D
 WButler
 5/23/90

Handwritten initials and signatures:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 25, 1990

Docket No. 50-278

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Director-Licensing, MC 5-2A-5
Philadelphia Electric Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
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Sincerely,

A handwritten signature in cursive script that reads "Gene Y. Suh".

Gene Y. Suh, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 156 to
License No. DPR-56
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. George A. Hunger, Jr.
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

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Peach Bottom Township
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 12, 1990, as supplemented on May 18, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

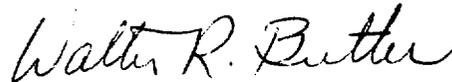
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 156, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 25, 1990

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 156, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 25, 1990

PDI-2/LA
M. O'Brien
5/21/90

PDI-2/PM
GSuh:mj
5/21/90

OGC
L. Dewey
5/23/90

PDI-2/D
WButler
5/23/90

WB

ATTACHMENT TO LICENSE AMENDMENT NO. 156

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Page

234a

PBAPS

LIMITING CONDITIONS FOR OPERATION

3.11.D. Shock Suppressors (Snubbers) on Safety Related Systems

3.11.D.1 During all modes of operation all snubbers on safety-related systems shall be operable except as noted in 3.11.D.2 and 3.11.D.3 below. Snubbers on non-safety related systems are excluded from this requirement if their failure or failure of the system on which installed has no adverse effect on a safety-related system.

3.11.D.2 During operation in the cold shutdown or refueling modes, snubbers located on systems required to be operable shall be operable except as noted in 3.11.D.3.

3.11.D.3 With one or more snubbers inoperable under the requirements of 3.11.D.1, within 72 hours, replace or restore the inoperable snubber to the operable status and perform an engineering evaluation per specification 4.11.D.6. If these requirements cannot be met, declare the supported system inoperable and follow the applicable Limiting Condition for Operation for that System.

SURVEILLANCE REQUIREMENTS

4.11.D. Shock Suppressors (Snubbers) on Safety Related Systems

4.11.D.1

Snubbers required to be operable under the provisions of 3.11.D.1 shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.6.G.

4.11.D.2

Snubbers required to be operable under the provisions of 3.11.D.1 shall be visually inspected according to the following schedule.

No. of Snubbers Found Inoperable During Inspection Period	Next Visual Inspection Period*
0	18 mo. \pm 25%
1	12 mo. \pm 25%
2	6 mo. \pm 25%
3,4	4 mo. \pm 25%
5,6,7	2 mo. \pm 25%
8 or more	1 mo. \pm 25%

The required inspection interval shall not be lengthened more than one step at a time. The provisions for extending surveillance frequency included in Section 1.0 Definitions do not apply. Snubbers may be categorized in two groups, "accessible" or "inaccessible", based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

*Those surveillances on inaccessible snubbers which must be performed on or before May 26, 1990 and are required by TS 4.11.D.2 may be delayed for a period not to exceed December 31, 1990.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING

AMENDMENT NO. 156 TO FACILITY OPERATING LICENSES NO. DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

DOCKET NO. 50-278

1.0 INTRODUCTION

By letter dated April 12, 1990, as supplemented on May 18, 1990, Philadelphia Electric Company requested an amendment to Facility Operating License No. DPR-56 for the Peach Bottom Atomic Power Station, Unit No. 3. The licensee's May 18, 1990 letter provided clarifying information related to the licensee's safety evaluation transmitted in its original April 12, 1990 submittal. The staff has determined that the supplemental information does not affect the proposed determination that the amendment involves no significant hazards consideration. This amendment would allow a one-time extension of about seven months for the performance of required visual inspections of inaccessible snubbers. As a result of previous snubber inspections, Unit 3 is currently required by TS 4.11.D.2 to visually inspect inaccessible snubbers at an interval of six months \pm 25%. The licensee is proposing that the next inspection due May 26, 1990 be postponed until the scheduled mid-cycle outage in the fourth quarter of 1990. The subject snubbers are inaccessible during reactor power operation and in the absence of the proposed TS change, plant shutdown would be required to perform the visual inspections.

2.0 EVALUATION

Technical Specification 4.11.D.2 requires that snubbers on safety-related components and piping be visually inspected at various intervals depending upon the snubber failures identified by the previous inspection. An increase in the number of failures would decrease the inspection interval.

During a visual inspection in January 1987, three snubbers out of a total population of 150 inaccessible snubbers were found with uncovered hydraulic fluid ports, indicating loss of fluid. This was attributed to seal failure and the snubbers were replaced with similar rebuilt snubbers. Based on three failures, TS 4.11.D.2 required the inspection interval to be reduced from the previous 18 months \pm 25% to four months \pm 25%. A subsequent inaccessible snubber visual inspection allowed the inspection interval to be lengthened to six months \pm 25%. During the period of

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March 31, 1987 to November 1989, Unit 3 did not operate. Most recently, prior to startup of Unit 3, 80% of the inaccessible snubbers were functionally tested during the period from May 1988 to October 1989 and verified operable and 100% of the inaccessible snubbers were visually inspected in October 1989 and confirmed to be free of discrepancies that could affect operability. These measures were in excess of TS requirements, and were undertaken to provide greater assurance that Unit 3 was starting up with an operable snubber population. However, the six month \pm 25% inspection interval was not increased due to the unit being shutdown since the last visual inspection. Therefore, the next visual inspection is due six months \pm 25% from the previous inspection or no later than May 26, 1990.

Although the proposed one-time extension of the visual inspection interval may result in a slight increase in the probability of a malfunction of the snubbers, the reduction in safety margin to components and piping runs is insignificant. This assessment is based on the fact that the malfunctions, if they occur, are likely to be randomly distributed and in general, the malfunction of a few snubbers in a large piping run is not likely to render the system inoperable. All piping runs and components whose structural integrity could be jeopardized by the failure of only a small number of snubbers have been evaluated individually. The High Pressure Coolant Injection (HPCI) system and the recirculation pumps fall in this category. All the snubbers on the HPCI system and the two recirculation pumps were included among the inaccessible snubbers which were functionally tested and verified operable prior to startup of Unit 3. This provides assurance that the overall increase in the probability of a malfunction or reduction in safety margin will not be significant.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (55 FR 17328) on April 24, 1990 and consulted with the Commonwealth of Pennsylvania. No public comments were received and the Commonwealth of Pennsylvania did not have any comments.

The staff has concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 25, 1990

Principal Contributor:

J. Rajan