UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 28, 1988

Docket No. 50-277, 278

Posted
Correction to
Amdt. 132 to DPR-56

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, PA 19101

Dear Mr. Bauer:

SUBJECT: CORRECTION LETTER FOR ERRORS IN LICENSE AMENDMENT NOS. 129 AND 132 UNITS 2 AND 3

RE: PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

On September 11, 1987 the Commission issued amendment number 123 for Unit 2 which made changes in the expression for the trip level setpoint for the APRM High Flux parameter on Technical Specification Table 3.1.1, page 37. The expression as modified read "(0.58W + 62 - 0.58 delta W)" multiplied by "FRP/MFLPD" with footnotes (12) and (13) being listed as applicable.

On March 3, 1988 the Commission issued amendment number 129 for Unit 2 which changed the main steamline high radiation setpoint in Table 3.1.1, page 38, and also made administrative changes to pages 37 and 38 of Table 3.1.1 to add a numbering sequence for the parameters in the table. In updating the previously provided copy of the Table 3.1.1, page 37, to reflect the amendment no. 129 changes the staff inadvertently left out part of the APRM High Flux trip level setting expression. Specifically, the portion "FRP/MFLPD" with notations to footnotes (12) and (13) was not included.

We are providing this letter to clarify that the expression "(0.58W + 62 - 0.58 delta W) FRP/MFLPD" with notations to footnotes (12) and (13) was not changed by amendment no. 129 to the Unit 2 facility license. Accordingly, the attached corrected Technical Specification page 37 is provided for amendment number 129.

Also, in amendment no. 129 for Unit 2 and amendment no. 132 issued for Unit 3 on March 3, 1988 the staff included page no. 47 of the BASES section of the Technical Specification for document completeness. The version of page 47 included in this amendment did not reflect changes to the page made by

amendment no. 65 to Unit 2 and amendment no. 64 to Unit 3 issued on March 26, 1980. Therefore, page no. 47 provided with the license amendment package of March 3, 1988 for both Units 2 and 3 should be replaced with the attached page 47 for both Units 2 and 3.

We regret any inconvenience this may have caused your staff.

Sincerely,

Robert Martin

Robert E. Martin, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures: Technical Specification
Pages 37 and 47 for Unit 2 and
Page 47 for Unit 3

cc w/enclosures
See next page

cc:

Mr. J. S. Kemper, Senior Vice President
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr., Esq.
1747 Pennsylvania Avenue, N.W.
Washington, D.C. 20006

Philadelphia Electric Company
ATTN: Mr. D. M. Smith, Vice President
Peach Bottom Atomic Power Station
Route 1, Box 208
Delta, Pennsylvania 17314

Mr. W. M. Alden
Engineer-In-Charge-Licensing
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

H. Chris Schwemm
Vice President, Production
Atlantic Electric
P. O. Box 1500
1199 Black Horse Pike
Pleasantville, New Jersey 08232

Resident Inspector
U. S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Mr. Bryan W. Gorman
Manager - External Affairs
Public Service Electric & Gas Company
P. O. Box 236, N28
Hancocks Bridge, New Jersey 08038

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Mr. Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Mr. Gary Mock
P. O. Box 09181
Columbus, Ohio 43209

Delmarva Power and Light Company
c/o Jack Urban
General Manager, Fuel Supply
800 King Street
P.O. Box 231
Wilmington, DE 19899

Mr. Tom Magette
Power Plant Research Program
Department of Natural Resources
B-3
Tawes State Office Building
Annapolis, Maryland 21401

Mr. Roland Fletcher
Department of Environment
201 West Preston Street
Baltimore, Maryland 21201

Amendment No. 25,34,42,70,78,123, 129 -37-

Table 3.1.1

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENTATION REQUIREMENT

Item	Minimum No. of Operable Instrument Channels per Trip System (1)	Trip Function	Trip Level Setting	Modes in which Function Must be Operable			Number of Instrument Channels Provided by Design	Action (1)
				Refuel (7)	Startup	Run		
1	1	Mode Switch In Shutdown		X	X	X	1 Mode Switch (4 Sections)	A
2	1	Manual Scram		X	X	X	2 Instrument Channels	A
3	3	IRM High Flux	≤120/125 of Full Scale	X	X	(5)	8 Instrument Channels	A
4	3	IRM Inoperative		X	X	(5)	8 Instrument Channels	A
5	2	APRM High Flux	$(0.58W+62-0.58\Delta W)$ / W			X	6 Instrument Channels	A or B
6	2	APRM Inoperative	(11)	X	X	X	6 Instrument Channels	A or B
7	2	APRM Downscale	≥2.5 Indicated on Scale			(10)	6 Instrument Channels	A or B
8	2	APRM High Flux in Startup	≤15% Power	X	X		6 Instrument Channels	A
9	2	High Reactor Pressure	≤1055 psig	X(9)	X	X	4 Instrument Channels	A
10	2	High Drywell Pressure	≤2 psig	X(8)	X(8)	X	4 Instrument Channels	A
11	2	Reactor Low Water Level	≥0 in. Indicated Level	X	X	X	4 Instrument Channels	A

3.1 BASES

The reactor protection system automatically initiates a reactor scram to:

1. Preserve the integrity of the fuel cladding.

2. Preserve the integrity of the reactor coolant system.

3. Minimize the energy which must be absorbed following a loss of coolant accident, and prevent inadvertant criticality.

This specification provides the limiting conditions for operation necessary to preserve the ability of the system to perform its intended function even during periods when instrument channels may be out of service because of maintenance. When necessary, one channel may be made inoperable for brief intervals to conduct required functional tests and calibrations.

The reactor protection system is of the dual channel type (Reference subsection 7.2 FSAR). The system is made up of two independent trip systems, each having two subchannels of tripping devices. Each subchannel has an input from at least one instrument channel which monitors a critical parameter.

The outputs of the subchannels are combined in a 1 out of 2 logic; i.e, an input signal on either one or both of the subchannels will cause a trip system trip. The outputs of the trip systems are arranged so that a trip on both systems is required to produce a reactor scram.

This system meets the intent of IEEE - 279 for Nuclear Power Plant Protection Systems. The system has a reliability greater than that of a 2 out of 3 system and somewhat less than that of a 1 out of 2 system.

With the exception of the Average Power Range Monitor (APRM) channels, the Intermediate Range Monitor (IRM) channels, the Main Steam Isolation Valve closure and the Turbine Stop Valve closure, each subchannel has one instrument channel. When the minimum condition for operation on the number of operable instrument channels per untripped protection trip system is met or if it cannot be met and the affected protection trip system is placed in a tripped condition, the effectiveness of the protection system is preserved.

The APRM instrument channels are provided for each protection trip system. APRM's A and E operate contacts in one subchannel and APRM's C and E operate contacts in the other subchannel. APRM's B, D and F are arranged similarly in

3.1 BASES

The reactor protection system automatically initiates a reactor scram to:

1. Preserve the integrity of the fuel cladding.

2. Preserve the integrity of the reactor coolant system.

3. Minimize the energy which must be absorbed following a loss of coolant accident, and prevent inadvertant criticality.

This specification provides the limiting conditions for operation necessary to preserve the ability of the system to perform its intended function even during periods when instrument channels may be out of service because of maintenance. When necessary, one channel may be made inoperable for brief intervals to conduct required functional tests and calibrations.

The reactor protection system is of the dual channel type (Reference subsection 7.2 FSAR). The system is made up of two independent trip systems, each having two subchannels of tripping devices. Each subchannel has an input from at least one instrument channel which monitors a critical parameter.

The outputs of the subchannels are combined in a 1 out of 2 logic; i.e, an input signal on either one or both of the subchannels will cause a trip system trip. The outputs of the trip systems are arranged so that a trip on both systems is required to produce a reactor scram.

This system meets the intent of IEEE - 279 for Nuclear Power Plant Protection Systems. The system has a reliability greater than that of a 2 out of 3 system and somewhat less than that of a 1 out of 2 system.

With the exception of the Average Power Range Monitor (APRM) channels, the Intermediate Range Monitor (IRM) channels, the Main Steam Isolation Valve closure and the Turbine Stop Valve closure, each subchannel has one instrument channel. When the minimum condition for operation on the number of operable instrument channels per untripped protection trip system is met or if it cannot be met and the affected protection trip system is placed in a tripped condition, the effectiveness of the protection system is preserved.

The APRM instrument channels are provided for each protection trip system. APRM's A and E operate contacts in one subchannel and APRM's C and E operate contacts in the other subchannel. APRM's B, D and F are arranged similarly in