

June 10, 1985

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Dockets Nos. 50-277
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Posted
Amatt. 112
to DPR-56

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 109 and 112 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments are in response to your request dated October 1, 1981. We previously issued Amendment No. 89 to Facility Operating License No. DPR-44 and Amendment No. 89 to Facility Operating License No. DPR-56 as partial response to your October 1, 1981 letter.

The changes to the Technical Specifications permit the following:

- 1) Changes in the "Remarks" column of Table 3.2.B to provide indication that an interlock applies to both the Residual Heat Removal (RHR) and Core Spray Systems.
- 2) A clarification pertaining to the Suppression Chamber High Level trip setting in accordance with Mark I containment studies (Table 3.2.B).
- 3) A revision concerning the surveillance requirements for the Primary Containment Isolation Signal (PCIS) and Low Pressure Coolant Injection (LPCI) interlock switch to correct an error by requiring more stringent requirements (Table 4.2.B).
- 4) Correct the calibration frequency for Reactor Level Instrumentation which had inadvertently been changed under previous amendments (Table 4.2.F).
- 5) A revision to Appendix B thermal mapping reporting frequency from 30 days to annually. In addition, this section is revised to more clearly specify the required events necessary to initiate thermal mapping monitoring and exempt data collection during periods when river and weather conditions preclude safe data gathering (Appendix B, Section 3.1).

Mr. Edward G. Bauer

-2-

The last remaining request in this application (Diesel Fuel Oil Quality) is being handled as a separate review.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

"ORIGINAL SIGNED BY:"

Gerald E. Gears, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 109 to DPR-44
2. Amendment No. 112 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

ORR #4: DL
Rindram
3/7/85

ORR #4: DL
Gears; cf
3/7/85

ORR #4: DL
JSC
3/12/85

ENER: DE
Saworth/Bills
4/17/85 3/12/85

OELT
L. Finkelman
3/17/85

AD: DL
L. Finkelman
6/10/85

chg. made
6/10/85

Mr. E. G. Bauer, Jr.
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

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Pennsylvania Department of
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Board of Supervisors
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R. D. #1
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 109
License No. DPR-44

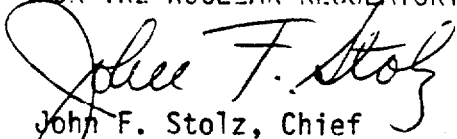
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated October 1, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 109, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
- Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 109

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendices "A" and "B" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove

Insert

Appendix A

69

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Appendix B

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TABLE 3.2.B (CONTINUED)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Minimum No. Of Operable Instrument Channels Per Trip System(1)	Trip Function	Trip Level Setting	Number of Instru- ment Channels Pro- vided by Design	Remarks
1	Core Spray Sparger to Reactor Pressure Vessel d/p	1 (+/-1.5) psid	2 Inst. Channels	Alarm to detect core spray sparger pipe break.
2	Condensate Storage Tank Low Level	>/= 5 feet above tank bottom	2 Inst. Channels	Provides interlock to HPCI pump suction valves.
2	Suppression Chamber High Level	</= 5 inches above torus midpoint	2 Inst. Channels	Transfers HPCI pump suction to suppression chamber.

TABLE 4.2.B
MINIMUM TEST AND CALIBRATION FREQUENCY FOR CSCS

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Reactor Water Level (7)	(1) (3)	Once/operating cycle	Once/day
2) Drywell Pressure (7)	(1) (3)	Once/operating cycle	Once/day
3) Reactor Pressure (7)	(1) (3)	Once/operating cycle	Once/day
4) Reactor Pressure - PCIS/LPCI Interlock	(1)	Once/3 months	None
5) Auto Sequencing Timers	NA	Once/operating cycle	None
6) ADS - LPCI or CS Pump Disch. Pressure Interlocks	(1)	Once/3 months	None
7) Trip System Bus Power Monitors	(1)	NA	None
8) Core Spray Sparger d/p	(1)	Once/6 months	Once/day
9) Steam Line High Flow (HPCI & RCIC)	(1)	Once/3 months	None
10) Steam Line High Flow Timers (HPCI and RCIC)	NA	Once/operating cycle	None
11) Steam Line High Temp. (HPCI & RCIC)	(1) (3)	Once/operating cycle	Once/day
12) Safeguards Area High Temp.	(1)	Once/3 months	None

TABLE 2.B (CONTINUED)
MINIMUM TEST AND CALIBRATION FREQUENCY FOR CSCS

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
13) HPCI and RCIC Steam Line Low Pressure	(1)	Once/3 months	None
14) HPCI Suction Source Levels	(1)	Once/3 months	None
15) 4KV Emergency Power System Voltage Relays (HGA,SV)	Once/operating cycle	Once/5 years	None
16) ADS Relief Valves Bellows Pressure Switches	Once/operating cycle	Once/operating cycle	None
17) LPCI/Cross Connect Valve Position	Once/refueling cycle	N/A	N/A
18) 4KV Emergency Power Source Degraded Voltage Relays (IAV,CV-6,ITE)	Once/month	Once/operating cycle	None

-81a-

TABLE (F

MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
1) Reactor Level	Once/6 months	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Water Temperature	Once/operating cycle** Once/6 months***	Once Each Day** Once Each Shift***
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/Operating cycle	Once/Month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (Acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle

** Effective when modification associated with this amendment request is complete.

***Delete when modification associated with this amendment request is complete.

2.0
2.1

PROTECTION LIMITS
Thermal (deleted)

3.0 MONITORING REQUIREMENTS

3.1 Thermal Objective

To assure that the water temperature in the Conowingo Pond is not increased to a level that adverse effects on the aquatic biota would result.

Specification

The bulk temperature of the condenser inlet water and of the water discharged to Conowingo Pond from the discharge canal shall be transmitted to the PBAPS control room and logged. Temperatures at Holtwood Dam and the Pennsylvania/Maryland state line shall be transmitted to the PBAPS control room and logged. For purposes of maintenance, the temperature monitoring system may be inoperable for a period not to exceed 7 days.

When less than three towers are in operation and the flow of the Susquehanna River through Conowingo Pond as calculated daily is less than 15,000 cfs for seven consecutive days with at least one unit operating, thermal plume mapping shall be performed at least once per week during the period of low flow, except in those cases where weather and river conditions preclude safe data collection. The thermal mapping shall be performed as follows: Isothermal plots of the receiving waters shall be produced at 1° F intervals. Data will be collected utilizing a motor boat mounted temperature recorder. The results of this thermal mapping along with applicable data on river flows, heat discharged from the station and the number of cooling towers in operation shall be compiled in individual or composite reports and shall be submitted, at least annually, to the NRC. Under these conditions, measurements of the bulk temperature of the water discharged to Conowingo Pond from the discharge canal



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-56

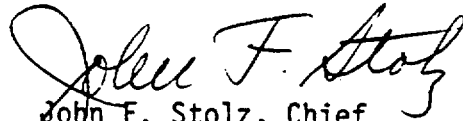
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated October 1, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 112, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 112

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendices "A" and "B" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove

Insert

Appendix A

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Appendix B

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TABLE 3.2.B (CONTINUED)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Minimum No. Of Operable Instrument Channels Per Trip System(1)	Trip Function	Trip Level Setting	Number of Instru- ment Channels Pro- vided by Design	Remarks
1	Core Spray Sparger to Reactor Pressure Vessel d/p	1 (+/-1.5) psid	2 Inst. Channels	Alarm to detect core spray sparger pipe break.
2	Condensate Storage Tank Low Level	>= 5 feet above tank bottom	2 Inst. Channels	Provides interlock to HPCI pump suction valves.
2	Suppression Chamber High Level	<= 5 inches above torus midpoint	2 Inst. Channels	Transfers HPCI pump suction to suppression chamber.

TABLE 4.2.B
MINIMUM TEST AND CALIBRATION FREQUENCY FOR CSCS

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Reactor Water Level (7)	(1) (3)	Once/operating cycle	Once/day
2) Drywell Pressure (7)	(1) (3)	Once/operating cycle	Once/day
3) Reactor Pressure (7)	(1) (3)	Once/operating cycle	Once/day
4) Reactor Pressure - PCIS/LPCI Interlock	(1)	Once/3 months	None
5) Auto Sequencing Timers	NA	Once/operating cycle	None
6) ADS - LPCI or CS Pump Disch. Pressure Interlocks	(1)	Once/3 months	None
7) Trip System Bus Power Monitors	(1)	NA	None
8) Core Spray Sparger d/p	(1)	Once/6 months	Once/day
9) Steam Line High Flow (HPCI & RCIC)	(1)	Once/3 months	None
10) Steam Line High Flow Timers (HPCI and RCIC)	NA	Once/operating cycle	None
11) Steam Line High Temp. (HPCI & RCIC)	(1) (3)	Once/operating cycle	Once/day
12) Safeguards Area High Temp.	(1)	Once/3 months	None

TAE 2.2.B (CONTINUED)
MINIMUM TEST AND CALIBRATION FREQUENCY FOR CSCS

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
13) HPCI and RCIC Steam Line Low Pressure	(1)	Once/3 months	None
14) HPCI Suction Source Levels	(1)	Once/3 months	None
15) 4KV Emergency Power System Voltage Relays (HGA,SV)	Once/operating cycle	Once/5 years	None
16) ADS Relief Valves Bellows Pressure Switches	Once/operating cycle	Once/operating cycle	None
17) LPCI/Cross Connect Valve Position	Once/refueling cycle	N/A	N/A
18) 4KV Emergency Power Source Degraded Voltage Relays (IAV,CV-6,ITE)	Once/month	Once/operating cycle	None

-81a-

TABLE 4.1

MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
1) Reactor Level	Once/6 months	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Water Temperature	Once/operating cycle** Once/6 months***	Once Each Day** Once Each Shift***
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/Operating cycle	Once/Month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (Acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle

** Effective when modification associated with this amendment request is complete.

***Delete when modification associated with this amendment request is complete.

2.0 PROTECTION LI TS
2.1 Thermal (deleted)

3.0 MONITORING REQUIREMENTS

3.1 Thermal
Objective

To assure that the water temperature in the Conowingo Pond is not increased to a level that adverse effects on the aquatic biota would result.

Specification

The bulk temperature of the condenser inlet water and of the water discharged to Conowingo Pond from the discharge canal shall be transmitted to the PBAPS control room and logged. Temperatures at Holtwood Dam and the Pennsylvania/Maryland state line shall be transmitted to the PBAPS control room and logged. For purposes of maintenance, the temperature monitoring system may be inoperable for a period not to exceed 7 days.

When less than three towers are in operation and the flow of the Susquehanna River through Conowingo Pond as calculated daily is less than 15,000 cfs for seven consecutive days with at least one unit operating, thermal plume mapping shall be performed at least once per week during the period of low flow, except in those cases where weather and river conditions preclude safe data collection. The thermal mapping shall be performed as follows: Isothermal plots of the receiving waters shall be produced at 1° F intervals. Data will be collected utilizing a motor boat mounted temperature recorder. The results of this thermal mapping along with applicable data on river flows, heat discharged from the station and the number of cooling towers in operation shall be compiled in individual or composite reports and shall be submitted, at least annually, to the NRC. Under these conditions, measurements of the bulk temperature of the water discharged to Conowingo Pond from the discharge canal



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING

AMENDMENTS NOS. 109 AND 112 TO FACILITY OPERATING LICENSES NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

Introduction

By letter dated October 4, 1981, the Philadelphia Electric Company, et al. (the licensee) proposed revisions to the Technical Specifications (TSs) appended to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units 2 and 3. The proposed amendments involve the revisions evaluated below.

Evaluation

1. Changes in the "Remarks" Column on Table 3.2.B

By letter dated March 28, 1984, the Commission issued Amendments Nos. 96 and 98 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom facility. By those amendments, the requested changes in the "Remarks" column on Table 3.2.B were made. However, the attached Safety Evaluation inadvertently failed to address these changes.

These changes provide the same description of interlock functions for both the Residual Heat Removal - Low Pressure Core Injection (RHR-LPCI) Pump Discharge Pressure Interlock and the Core Spray Pump Discharge Pressure Interlock, namely, deferring actuation of the Automatic Depressurization System (ADS) until there is confirmation of Low Pressure Core Cooling System operation. These two interlock systems are designed to defer ADS actuation but the description originally found in the Table 3.2.B did not clearly indicate that this was the function of both interlocks. Therefore, this change provides clarification of the functions of both interlocks, and we conclude that this change is acceptable.

2. Clarification Pertaining to the Suppression Chamber
High Level Trip Setting

This requested change would permit the Suppression Chamber High Level trip setting (Table 3.2.B) to read "less than or equal to 5 inches above torus midpoint" rather than "less than or equal to 5 inches above normal water

level." By letter dated February 4, 1985, the licensee provided documentation of its telephone conversation with the NRC staff on December 14, 1984, which resulted in additional background information concerning this request. This change provides for new nomenclature for the setting involving the Suppression Chamber High Level trip as a result of modifications performed in accordance with the NRC Mark I Containment Program as described in NUREG-0661 ("Safety Evaluation Report - Mark I Containment Long-Term Program" - July 1980). The original suppression chamber (torus) "normal water level" was at elevation 110'0" or 15'6" above the bottom of the torus. Elevation 110'0" is also the torus midpoint (FSAR, Figure 5.2.1). The "current normal water levels" resulting from the Mark I Program are based upon the torus inventory limits specified in Section 3.7.A.1 of the current TSs. The minimum water volume (122,900 ft³) specification yields a lower limit of 14.5 ft for the "normal water level" and the maximum water volume (127,300 ft³) yields an upper limit of 14.9 ft for the "normal level." These revisions in the "normal levels" due to the Mark I Containment Program had the effect of inadvertently lowering the Suppression Chamber High Level trip setting which currently references in the TSs "normal water level" rather than a fixed physical point on the torus.

The Suppression Chamber High Level trip initiates transfer of the High Pressure Coolant Injection System (HPCI) suction from the condensate storage tank to the torus (suppression chamber). The purpose is to limit the rise in torus water inventory during a Loss-of-Coolant Accident (LOCA) to ensure an adequate gas space above the torus water level. Torus gas space insures proper venting of non-condensable gases through the vacuum breakers to drywell. The design basis of the original trip setting was the original torus (Suppression Chamber) normal water level (elevation 110'0" or 15'6" above the bottom of the torus). As indicated above, the torus midpoint is also at elevation 110'0" (or 15'6" above torus bottom).

The proposed TS change clarifies that the trip setpoint is tied to a physical location on the torus (i.e., torus midpoint which is elevation 110'0") and reconfirms the original design basis of this setpoint. This proposed change does not represent a change in either the normal torus water level established for the Mark I Program or a change in the original design basis for the trip setpoint. Since the requested change deals only with the nomenclature for designating the trip setpoint, we find that this change is acceptable without further evaluation.

3. Revision Concerning the Surveillance Requirements for the Primary Containment Isolation Signal (PCIS) and Low Pressure Coolant Injection (LPCI) Interlock Switch

The surveillance requirements for the reactor pressure instrumentation, which provides interlocks for several safety functions, were revised in Table 4.2.B (Minimum Test and Calibration Frequency for CSCS) as part of a requested revision authorized by Amendments Nos. 68/67. These amendments changed the calibration frequency from once every 3 months to once every operating cycle. The basis for this change was the installation of new analog transmitters. The three reactor pressure instrumentation interlock systems are shown in Table 3.2.B of the TSs (Reactor Low Pressure). The new analog transmitters as described in Amendments 68/67 were associated with only two of the three interlock systems. The instrumentation for the third reactor

low pressure interlock (PCIS/LPCI interlock) was not modified to employ new analog transmitters and, therefore, the surveillance frequency should remain at once every 3 months. To correct this error, the licensee proposes to add to Table 4.2.B, as a separate entry, the above discussed PCIS/LPCI interlock feature and require a calibration frequency for this interlock of once per 3 months. This action would reestablish the more stringent surveillance requirement which had been previously required by the Peach Bottom TSs prior to issuance of Amendments 68/67. The surveillance requirements which the NRC staff authorized for the new analog transmitters in Amendments 68/67 are not affected by this change. We conclude that this change is acceptable because it corrects an error in the TSs and results in reestablishment of surveillance requirements previously occurring in the Peach Bottom TSs.

4. Correction of an Error in the Calibration Frequency for Reactor Level Instrumentation in Table 4.2.F (Minimum Test and Calibration Frequency for Surveillance Instrumentation)

By Amendments Nos. 68/67, the calibration frequency for the Reactor Level Instrumentation in Table 4.2.F was lengthened from once per 6 months to once per operating cycle (approximately once per 18 months). The licensee states that the lengthening authorized by the above amendments was inadvertent and the proposed change would correct this error by reestablishing the original TS calibration frequency requirement of once per 6 months. Notwithstanding the inadvertent lengthening of the calibration frequency authorized by the above previous amendments, the licensee has stated that the shorter calibration frequency (once per 6 months) has continued to be used. We find that the proposed change is acceptable since it corrects an error and restores in the TSs the original required calibration frequency for Reactor Level Instrumentation.

5. Revision to Appendix B (Environmental Technical Specifications) Thermal Mapping Reporting Frequency and Clarification on the Required Events Necessary to Initiate Thermal Mapping

The first requested change involves a revision in the reporting frequency for thermal mapping results. The current Appendix B TSs require that any thermal mapping results be reported to the NRC within 30 days. The requested change would still require that thermal mapping results be compiled but that reports to the NRC be submitted at least annually rather than within 30 days. In assessing this requested change, we have reviewed the requirements and objectives of ETS Section 3.1 (Thermal Mapping) and the previous NRC actions involving these requirements, including Amendments 67/66 (April 24, 1980) and Amendments 92/94 (February 24, 1984) which were directly related to the Section. This above change does not alter the procedures or conditions for thermal mapping in ETS Section 3.1 but rather affects only the reporting frequency. We conclude that the proposed modification does not alter the objective of this ETS requirement which is to assure that thermal regimes, during cooling tower outages linked with extreme environmental conditions, are not detrimental to aquatic biota.

In addition, the licensee requested a modification of ETS 3.1 to preclude the collection of thermal mapping data by boat when weather and river conditions are not conducive to safe data collection. We believe that this is a reasonable modification which would enhance the health and safety of the licensee's employees and which would not significantly alter the objectives of the ETS as stated above. It should also be noted that ETS 3.1 also requires that temperature measurements downstream from the site at the Pennsylvania/Maryland state line "...be transmitted to the PBAPS control room and logged." Thus, there is continuous monitoring and recording of thermal regimes required by ETS 3.1 which remain unchanged by this request. In addition, the supplemental thermal mapping by boat as specified in ETS 3.1 would only be precluded if weather or river conditions would not allow safe data collection.

Therefore, based upon the above, we conclude that there will be no environmental impact attributable to the proposed action. These changes to the environmental monitoring (surveillance) requirements do not involve any change in station design or operation or involve an increase in effluent types or quantities, or increase in individual or cumulative occupational radiation exposure. The impact of the overall station operation has already been predicted and described in the Commission's FES for Peach Bottom Atomic Power Station, Units 2 and 3. Our review has not revealed impacts greater than those previously anticipated.

Environmental Consideration

These amendments involve changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. These amendments also relate to changes in recordkeeping, reporting, or administrative procedures or requirements. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 10, 1985

Principal Contributor: G. Gears