

REGULATORY DIVISION - PLEASE USE COPY



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20545

October 28, 1980

Dockets Nos. 50-277
and 50-278

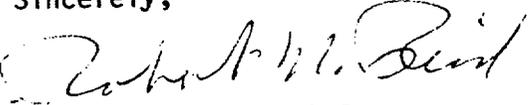
Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 74 and 73 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 15, 1980.

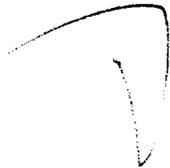
The changes to the TSs involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern 1) Emergency Power Supply/Inadequate Core Cooling, 2) Valve Position Indication, 3) Containment Isolation, 4) Shift Technical Advisor Augmentation, 5) Integrity of Systems Outside Containment, and 6) Iodine Monitoring.

Copies of our Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

- Enclosures:
1. Amendment No. 74 to DPR-44
 2. Amendment No. 73 to DPR-56
 3. Safety Evaluation
 4. Notice

cc w/enclosures: See next page



8011070125

Philadelphia Electric Company

cc w/enclosure(s):

Eugene J. Bradley
Philadelphia Electric Company
Assistant General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr.
1747 Pennsylvania Avenue, N.W.
Washington, D. C. 20006

Raymond L. Hovis, Esq.
35 South Duke Street
York, Pennsylvania 17401

Warren K. Rich, Esq.
Assistant Attorney General
Department of Natural Resources
Annapolis, Maryland 21401

Philadelphia Electric Company
ATTN: Mr. W. T. Ullrich
Peach Bottom Atomic
Power Station
Delta, Pennsylvania 17314

Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Curt Cowgill
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Director, Technical Assessment
Division
Office of Radiation Programs
(AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Region III Office
ATTN: EIS COORDINATOR
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

M. J. Cooney, Superintendent
Generation Division - Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Government Publications Section
State Library of Pennsylvania
Education Building
Commonwealth and Walnut Streets
Harrisburg, Pennsylvania 17126

cc w/enclosure(s) & incoming dtd.:

9/15/80

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. DPR-44

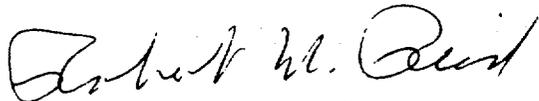
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated September 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 74, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Robert W. Reid".

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 28, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Page 85 was not changed; it is included for convenience only.

<u>Remove Page</u>	<u>Insert Page</u>
iii	iii
77	77
78	78
85	85
86	86
245	245
246	246
--	263 (new page)

PBAPS

TABLE OF CONTENTS (cont'd)

<u>LIMITING CONDITIONS FOR OPERATION</u>	<u>SURVEILLANCE REQUIREMENTS</u>	<u>PAGE NO.</u>
3.14 FIRE PROTECTION	4.14	240c
A. Water Fire Protection System	A	240c
B. CO2 Fire Protection System	B	240g
C. Fire Detection	C	240i
D. Fire Barrier Penetrations	D	240j
5.0 MAJOR DESIGN FEATURES		241
6.0 ADMINISTRATIVE CONTROLS		243
6.1 Responsibility		243
6.2 Organization		243
6.3 Facility Staff Qualifications		246
6.4 Training		246
6.5 Review and Audit		246
6.6 Reportable Occurrence Action		253
6.7 Safety Limit Violation		253
6.8 Procedures		253
6.9 Reporting Requirements		254
6.10 Record Retention		260
6.11 Radiation Protection Program		261
6.12 Fire Protection Inspections		261
6.13 High Radiation Area		262
6.14 Integrity of Systems Outside Containment		263
6.15 Iodine Monitoring		263
6.16 Environmental Qualification		264

TABLE 3.2.F

SURVEILLANCE INSTRUMENTATION

Minimum No. of Operable Instrument Channels	Instrument	Type Indication and Range	Action
2	Reactor Water Level	Recorder 0-60" Indicator 0-60"	(6) (7)
2	Reactor Pressure	Recorder 0-1500 psig Indicator 0-1200 psig	(1) (2) (3)
2	Drywell Pressure	Recorder 0-70 psig	(1) (2) (3)
2	Drywell Temperature	Recorder 0-400 °F Indicator 0-400 °F	(1) (2) (3)
2	Suppression Chamber Water Temperature	Recorder 0-600 °F Indicator 0-400 °F	(1) (2) (3)
2	Suppression Chamber Water Level	Recorder 0-2 ft. Indicator 0-2 ft.	(1) (5)
1	Control Rod Position	28 Volt Indicating Lights))) (1) (2) (3) (4)
1	Neutron Monitoring	SRM, IRM, LPRM 0-100%))
1	Safety-Relief Valve Position Indication	Acoustic or thermocouple	(8)

PBAPS

NOTES FOR TABLE 3.2.F

- 1) From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.
- 2) From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.
- 3) If the requirements of notes (1) and (2) cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold condition within 24 hours.
- 4) These surveillance instruments are considered to be redundant to each other.
- 5) In the event that all indications of this parameter are disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.
- 6) With the number of operable channels less than the minimum number of instrumentation channels shown in Table 3.2.F, either restore the inoperable channel to an operable status within 7 days, or be in at least hot shutdown within the next 12 hours.
- 7) If this parameter is not indicated in the control room, either restore at least one inoperable channel to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.
- 8) If this parameter is not indicated in the control room, either restore at least one channel to operable status within thirty days or be in at least hot shutdown within the next 12 hours.

TABLE 4.2.E

MINIMUM TEST AND CALIBRATION FREQUENCY FOR DRYWELL LEAK DETECTION

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Equipment Drain Sump Flow Integrator	(1)	Once/3 months	Once/day
2) Floor Drain Sump Flow Integrator	(1)	Once/3 months	Once/day
3) Air Sampling System	(1)	Once/3 months	Once/day

PBAPS

TABLE 4.2.F

UNIT 2

MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION		
<u>Instrument Channel</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Reactor Level	Once/operating cycle	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Temperature	Once/6 months	Once Each Shift
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle

TABLE 4.2.F

UNIT 2

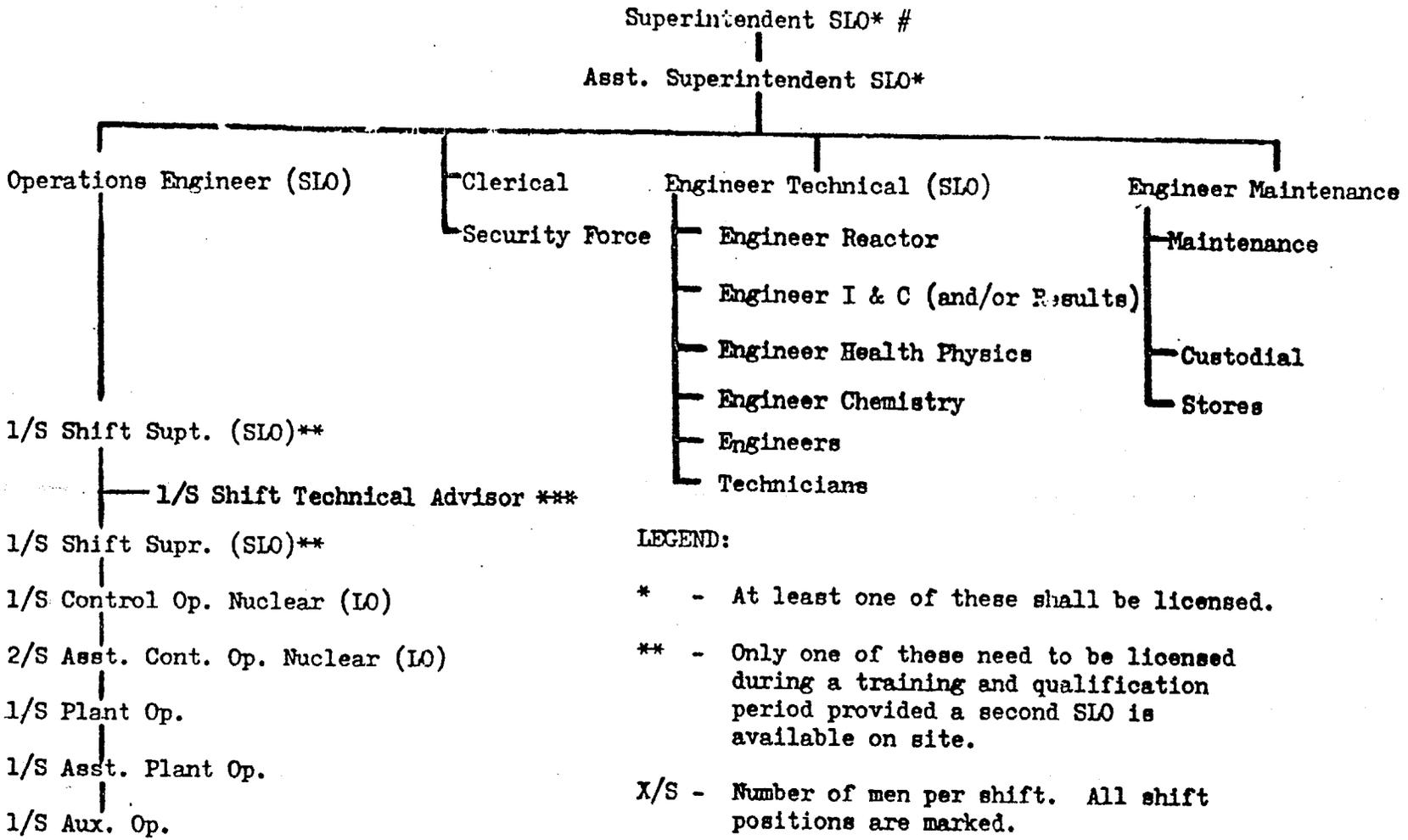
MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

<u>Instrument Channel</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Reactor Level	Once/operating cycle	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Temperature	Once/6 months	Once Each Shift
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle

Amendment No. 10, 39, 67, 74

245



LEGEND:

- * - At least one of these shall be licensed.
- ** - Only one of these need to be licensed during a training and qualification period provided a second SLO is available on site.
- X/S - Number of men per shift. All shift positions are marked.
- LO - NRC Licensed Operator
- SLO - NRC Licensed Senior Operator
- # - Responsible for onsite fire protection activities
- *** - Not applicable with plant in the shutdown or refuel mode.

ORGANIZATION FOR CONDUCT OF PLANT OPERATIONS

Figure 6.2-2

PBAPS

6.3 Facility Staff Qualifications

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Engineer-Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Station Superintendent and shall meet the requirements of Section 5.5 of ANSI N13.1-1971 and 10 CFR 55, Appendix A.

6.4.2 A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months.

6.5 Review and Audit

6.5.1 Plant Operation Review Committee (PORC)

Function

6.5.1.1 The Plant Operation Review Committee shall function to advise the Station Superintendent on all matters related to nuclear safety.

Composition

6.5.1.2 The Plant Operation Review Committee shall be composed of the:

Station Superintendent-Chairman
Station Assistant Superintendent
Engineer - Technical
Engineer - Maintenance
Engineer - Operations
Engineer - Results
Engineer - Reactor
Engineer - Instrument & Control
Engineer - Health Physics
Engineer - Chemistry
Shift Superintendent

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

PBAPS

6.14 Integrity of Systems Outside Containment

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following;

- 1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and
- 2) System leak test requirements, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Residual Heat Removal, (2) Core Spray, (3) Reactor Water Cleanup, (4) HPCI, and (5) RCIC.

6.15 Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas* under accident conditions. This program shall include the following:

1. Training of personnel,
- 2, Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

* Areas requiring personnel access for establishing hot shutdown condition.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-56

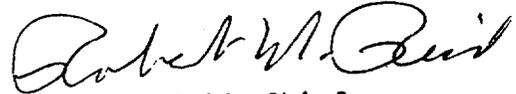
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated September 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 28, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Page 85 was not changed; it is included for convenience only.

<u>Remove Page</u>	<u>Insert Page</u>
iii	iii
77	77
78	78
85	85
86	86
245	245
246	246
--	263 (new page)

PBAPS

TABLE OF CONTENTS (cont'd)

<u>LIMITING CONDITIONS FOR OPERATION</u>	<u>SURVEILLANCE REQUIREMENTS</u>	<u>PAGE NO.</u>
3.14 FIRE PROTECTION	4.14	240c
A. Water Fire Protection System	A	240c
B. CO2 Fire Protection System	B	240g
C. Fire Detection	C	240i
D. Fire Barrier Penetrations	D	240j
5.0 MAJOR DESIGN FEATURES		241
6.0 ADMINISTRATIVE CONTROLS		243
6.1 Responsibility		243
6.2 Organization		243
6.3 Facility Staff Qualifications		246
6.4 Training		246
6.5 Review and Audit		246
6.6 Reportable Occurrence Action		253
6.7 Safety Limit Violation		253
6.8 Procedures		253
6.9 Reporting Requirements		254
6.10 Record Retention		260
6.11 Radiation Protection Program		261
6.12 Fire Protection Inspections		261
6.13 High Radiation Area		262
6.14 Integrity of Systems Outside Containment		263
6.15 Iodine Monitoring		263
6.16 Environmental Qualification		264

TABLE 3.2.F

SURVEILLANCE INSTRUMENTATION

Minimum No. of Operable Instrument Channels	Instrument	Type Indication and Range	Action
2	Reactor Water Level	Recorder 0-60" Indicator 0-60"	(6) (7)
2	Reactor Pressure	Recorder 0-1500 psig Indicator 0-1200 psig	(1) (2) (3)
2	Drywell Pressure	Recorder 0-70 psig	(1) (2) (3)
2	Drywell Temperature	Recorder 0-400 ^o F Indicator 0-400 ^o F	(1) (2) (3)
2	Suppression Chamber Water Temperature	Recorder 0-600 ^o F Indicator 0-400 ^o F	(1) (2) (3)
2	Suppression Chamber Water Level	Recorder 0-2 ft. Indicator 0-2 ft.	(1) (5)
1	Control Rod Position	28 Volt Indicating Lights))	(1) (2) (3) (4)
1	Neutron Monitoring	SRM, IRM, LPRM 0-100%))	
1	Safety-Relief Valve Position Indication	Acoustic or thermocouple	(8)

PBAPS

NOTES FOR TABLE 3.2.F

- 1) From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.
- 2) From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.
- 3) If the requirements of notes (1) and (2) cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold condition within 24 hours.
- 4) These surveillance instruments are considered to be redundant to each other.
- 5) In the event that all indications of this parameter are disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.
- 6) With the number of operable channels less than the minimum number of instrumentation channels shown in Table 3.2.F, either restore the inoperable channel to an operable status within 7 days, or be in at least hot shutdown within the next 12 hours.
- 7) If this parameter is not indicated in the control room, either restore at least one inoperable channel to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.
- 8) If this parameter is not indicated in the control room, either restore at least one channel to operable status within thirty days or be in at least hot shutdown within the next 12 hours.

TABLE 4.2.E

MINIMUM TEST AND CALIBRATION FREQUENCY FOR DRYWELL LEAK DETECTION

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Equipment Drain Sump Flow Integrator	(1)	Once/3 months	Once/day
2) Floor Drain Sump Flow Integrator	(1)	Once/3 months	Once/day
3) Air Sampling System	(1)	Once/3 months	Once/day

TABLE 4.2.F

UNIT 3

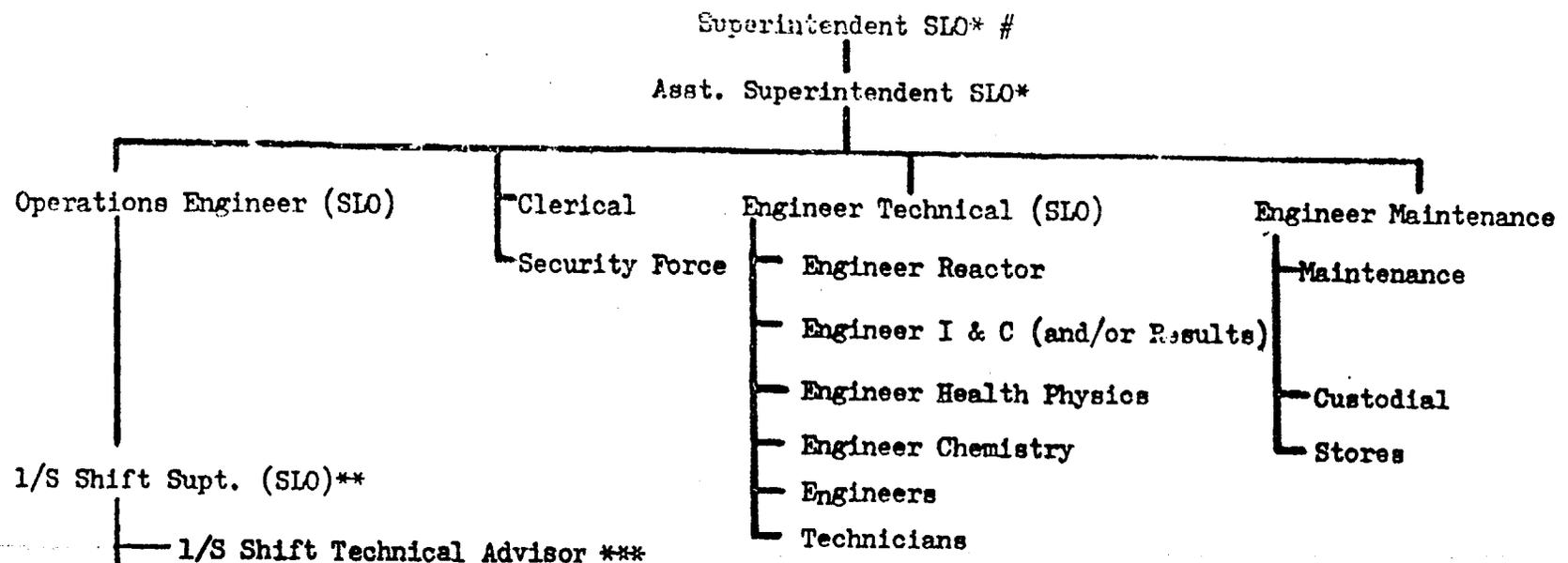
MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

<u>Instrument Channel</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) **Reactor Level	Once/operating cycle	Once Each Shift
*Reactor Level	Once/6 months	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Temperature	Once/6 months	Once Each Shift
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA***	Once/month
11) Safety Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA***	Once/month

* Deleted when modifications authorized by Amendment No. 67 are completed.

** Effective when modifications authorized by Amendment No. 67 are completed.

*** Perform instrument functional check once per operating cycle.



LEGEND:

- * - At least one of these shall be licensed.
- ** - Only one of these need to be licensed during a training and qualification period provided a second SLO is available on site.
- X/S - Number of men per shift. All shift positions are marked.
- LO - NRC Licensed Operator
- SLO - NRC Licensed Senior Operator
- # - Responsible for onsite fire protection activities
- *** - Not applicable with plant in the shutdown or refuel mode.

ORGANIZATION FOR CONDUCT OF PLANT OPERATIONS

Figure 6.2-2

PBAPS

6.3 Facility Staff Qualifications

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Engineer-Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Station Superintendent and shall meet the requirements of Section 5.5 of ANSI N13.1-1971 and 10 CFR 55, Appendix A.

6.4.2 A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months.

6.5 Review and Audit

6.5.1 Plant Operation Review Committee (PORC)

Function

6.5.1.1 The Plant Operation Review Committee shall function to advise the Station Superintendent on all matters related to nuclear safety.

Composition

6.5.1.2 The Plant Operation Review Committee shall be composed of the:

Station Superintendent-Chairman
Station Assistant Superintendent
Engineer - Technical
Engineer - Maintenance
Engineer - Operations
Engineer - Results
Engineer - Reactor
Engineer - Instrument & Control
Engineer - Health Physics
Engineer - Chemistry
Shift Superintendent

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

PBAPS

6.14 Integrity of Systems Outside Containment

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

- 1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and
- 2) System leak test requirements, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Residual Heat Removal, (2) Core Spray, (3) Reactor Water Cleanup, (4) HPCI, and (5) RCIC.

6.15 Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas* under accident conditions. This program shall include the following:

1. Training of personnel,
- 2, Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

* Areas requiring personnel access for establishing hot shutdown condition.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20585

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMENDMENT NO. 74 TO LICENSE NO. DPR-44
AND
AMENDMENT NO. 73 TO LICENSE NO. DPR-56

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

PHILADELPHIA ELECTRIC COMPANY

DOCKETS NOS. 50-277 AND 50-278

I. INTRODUCTION

By letter dated September 15, 1980, the Philadelphia Electric Company (the licensee) proposed changes to the Technical Specifications (TSs) appended to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. The changes involve the incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. The licensee's request is in direct response to the NRC staff's letter dated July 2, 1980.

II. BACKGROUND INFORMATION

By our letter dated September 13, 1979, we issued to all operating nuclear power plants requirements established as a result of our review of the TMI-2 accident. Certain of these requirements, designated Lessons Learned Category "A" requirements, were to have been completed by the licensee prior to any operation subsequent to January 1, 1980. Our evaluation of the licensee's compliance with these Category "A" items was attached to our letter to Philadelphia Electric Company dated February 26, 1980.

In order to provide reasonable assurance that operating reactor facilities are maintained within the limits determined acceptable following the implementation of the TMI-2 Lessons Learned Category "A" items, we requested that licensees amend their TSs to incorporate additional Limiting Conditions of Operation and Surveillance Requirements, as appropriate. This request was transmitted to all licensees on July 2, 1980. Included therein were model specifications that we had determined to be acceptable. The licensee's application is in direct response to our request. Each of the issues identified by the NRC staff and the licensee's response is discussed in the Evaluation below.

III. EVALUATION

1. Emergency Power Supply/Inadequate Core Cooling

As applicable to Boiling Water Reactors (BWRs), we indicated that water level instrumentation is important to post-accident monitoring and that surveillance

of this instrumentation should be performed. The licensee's response to this request stated that the current surveillance requirements for the reactor water level instrumentation at Peach Bottom is more conservative than our guidance. Specifically, instrument checks at Peach Bottom are performed once per shift instead of once per month. The licensee's application did include a proposed revision to bring the operability requirements into agreement with our guidelines. These guidelines, simply stated, require (1) two operable instrument channels; (2) with less than two channels operable, operability of two channels must be restored within seven days or reactor shutdown is required; (3) with less than one channel operable, operability must be restored within 48 hours. The licensee's application is consistent with these guidelines. Therefore, we find the proposed revision to be acceptable.

2. Valve Position Indication

Our requirements for installation of a reliable position indicating system for relief and safety valves were based on the need to provide the operator with a diagnostic aid to reduce the ambiguity between indications that might indicate either an open relief/safety valve or a small line break. Such a system did not need to be safety grade provided that backup methods of determining valve position are available. Since the indicating system provides no automatic action, the licensee proposed that limiting conditions for operation in the event of an inoperable channel are not appropriate and that the TSs should be limited to surveillance requirements. The licensee presented a discussion of the safety significance of this valve position indicating system and discussed alternate methods for diagnosing valve failure. We have reviewed the licensee's submittal and agree with his basic premise that there are a number of alternate backup methods for determining that a valve is open. However, these alternate methods would not provide indications that a valve has reseated. Therefore, we suggested that the TSs should require at least a primary or backup system of valve position indication to be operable or the reactor should be shutdown after 30 days. A 30-day limit is consistent with current practices for post-accident monitoring instrumentation. Accordingly, we find the licensee's submittal as modified by the NRC staff and agreed to by the licensee to be acceptable.

3. Containment Isolation

Our request indicated that the Specifications should include a Table of Containment Isolation Valves which reflect the diverse isolation signal requirement of this Lessons Learned issue. The licensee's response stated that his application dated July 16, 1980, was responsive to this request.

The licensee's request dated July 16, 1980, concerns the addition of isolation valves in the instrument nitrogen compressor suction line and the radioactive gas sampler line to ensure redundant isolation. This application is under review and will be processed separately. However, we have reviewed existing

Table 3.7.1 of the Peach Bottom Specifications which reflects diverse isolation signals to each valve. Therefore, we conclude that no further change is required.

4. Shift Technical Advisor (STA)

Our request indicated that the TSs related to minimum shift manning should be revised to reflect the augmentation of a STA. The licensee's application would add one STA to each shift to perform the function of accident assessment. The individual performing this function will have at least a bachelor's degree or equivalent in a scientific or engineering discipline with special training in plant design, and response and analysis of the plant for transients and accidents. Part of the STA duties are related to operating experience review function. Based on our review, we find the licensee's submittal to satisfy our requirements and is acceptable.

5. Integrity of Systems Outside Containment

Our request indicated that licensees should be required to periodically conduct a System Integrity Measurements Program to prevent the release of significant amounts of radioactivity to the environment via leakage from engineered safety systems and auxiliary systems which are located outside reactor containment. The licensee's program includes provisions for a preventive maintenance program and periodic visual inspections. The program also includes system leak test measurements at frequencies not to exceed refueling cycle intervals.

Based on our review we find that inclusion of this requirement in the Administrative Controls Section of the TSs satisfies our requirement and is acceptable.

6. Iodine Monitoring

Our request indicated that the licensees should implement a program which will ensure the capability to determine the airborne iodine concentration in areas requiring personnel access under accident conditions. The licensee's program includes training of personnel, procedures for monitoring and provisions for maintenance of sampling and analysis equipment.

Based on our review we find that inclusion of this requirement in the Administrative Controls Section of the TSs satisfies our requirement and is acceptable.

IV. ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

V. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 28, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

7590-01

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY, ET AL

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 74 and 73 to Facility Operating Licenses Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The amendments to the Technical Specifications involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern 1) Emergency Power Supply/Inadequate Core Cooling, 2) Valve Position Indication, 3) Containment Isolation, 4) Shift Technical Advisor Augmentation, 5) Integrity of Systems Outside Containment, and 6) Iodine Monitoring.

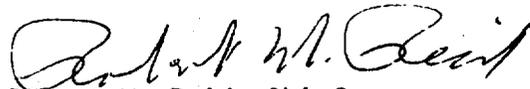
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated September 15, 1980, (2) Amendments Nos. 74 and 73 to Licenses Nos. DPR-44 and DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, DC, and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of October 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing