



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

November 19, 1980

Dockets Nos. 50-277  
and 50-278

Mr. Edward G. Bauer, Jr.  
Vice President and General Counsel  
Philadelphia Electric Company  
2301 Market Street  
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 75 and 74 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated August 8, 1980, as supplemented by your letter dated October 3, 1980.

The amendments add operability and surveillance requirements for the new seismic monitoring instrumentation system you are planning to install.

Your application stated that the new system should be operational by the fall of 1980. In the event that you expect any delay in installation beyond December 1, 1980, we request that you advise us promptly.

Copies of our Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 75 to DPR-44
  2. Amendment No. 74 to DPR-56
  3. Safety Evaluation
  4. Notice
- cc w/enclosures: See next page

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Philadelphia Electric Company

cc w/enclosure(s):

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Government Publications Section  
State Library of Pennsylvania  
Education Building  
Commonwealth and Walnut Streets  
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cc w/enclosure(s) & incoming dtd.:  
8/8/80 & 10/3/80

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75  
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 8, 1980, as supplemented October 3, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

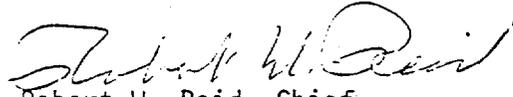
Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

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Insert Pages

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vii (new page)

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PBAPS

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4.15	Seismic Monitoring Instrumentation Surveillance Requirements	240p

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.15 Seismic Monitoring  
InstrumentationApplicability

Applies to the operational status of the seismic monitoring instrumentation.

Specifications

- A. The seismic monitoring instrumentation shown in Table 3.15 shall be operable.
- B. With one or more seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.2, prepare and submit a Special Report to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within the next 10 working days outlining the cause of the malfunction and the plans for restoring the instrument(s) to operable status.
- C. The provisions of Specification 3.0.c are not applicable.

4.15 Seismic Monitoring  
InstrumentationApplicability

Applies to the surveillance requirements of the seismic monitoring instrumentation.

Specifications

- A. Each of the required seismic monitoring instruments shall be demonstrated operable by the performance of the Instrument Check, Instrument Functional Test, and Instrument Calibration operations at the frequencies shown in Table 4.15.
- B. Each of the required seismic monitoring instruments actuated during a seismic event shall be restored to operable status within 24 hours and an Instrument Calibration performed within 5 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. In lieu of any other report required by Specification 6.9.2, a Special Report shall be prepared and submitted to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within 10 working days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.

TABLE 3.15\*\*

SEISMIC MONITORING INSTRUMENTATION

<u>Instruments and Sensor Locations#</u>	<u>Measurement Range</u>	<u>Minimum Instruments Operable</u>
1. Triaxial Time-History Accelerographs		
a. Containment Foundation (torus compartment)	0.1-10g	1
b. Refueling Floor	0.1-10g	1
c. RCIC Pump (Rm #7)	0.1-10g	1
d. "C" Diesel Generator	0.1-10g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01-2g	1
b. Refueling Floor	0.01-2g	1
c. "C" Diesel Generator	0.01-2g	1
3. Triaxial Response-Spectrum Recorders		
a. Cable Spreading Rm	0.1-10g	1*

\* With reactor control room annunciation

\*\* Effective upon completion of installation.

# Seismic instrumentation located in Unit 2

TABLE 4.15\*\*

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instruments and Sensor Locations#</u>	<u>Instrument*</u> <u>Check</u>	<u>Instrument*</u> <u>Functional</u> <u>Test</u>	<u>Instrument*</u> <u>Calibration</u>
<b>1. Triaxial Time-History Accelerographs</b>			
a. Containment Foundation (torus compartment)	M	SA	R
b. Refueling Floor	M	SA	R
c. PCIC Pump (Rm #7)	M	SA	R
d. "C" Diesel Generator	M	SA	R
<b>2. Triaxial Peak Accelerographs</b>			
a. Reactor Piping (Drywell)	NA	NA	R
b. Refueling Floor	NA	NA	R
c. "C" Diesel Generator	NA	NA	R
<b>3. Triaxial Response-Spectrum Recorders</b>			
a. Cable Spreading Rm	M	SA	R

\* Surveillance Frequencies

M: every month  
 SA: every 6 months  
 R: every 18 months

\*\* Effective upon completion of installation.

# Seismic instrumentation located in Unit 2.

PBAPS

3.15/4.14 BASES

The operability of the seismic monitoring instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the plant.

The time-history recordings of the triaxial time-history accelerographs are done in the cable spreading room on a digital cassette accelerograph. In addition to being recorded, the containment foundation sensor is analyzed on line by a response spectrum analyzer. The spectrum of any sensor can be obtained by playing back its time-history cassette through the response spectrum analyzer.

## 6.9.2 Continued

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- (4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 Unique Reporting Requirements

Special reports shall be submitted to the Director of the appropriate Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Loss of shutdown margin, Specification 3.3.A and 4.3.A within 14 days of the event.
- b. Reactor vessel inservice inspection, Specification 3.6.G and 4.6.G within 90 days of the completion of the reviews.
- c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.
- d. Primary containment leak rate testing approximately three months after the completion of the periodic integrated leak rate test (Type A) required by Specification 4.7.A.2.c.2. For each periodic test, leakage test results from Type A, B and C tests shall be reported. B and C tests are local leak rate tests required by Specification 4.7.A.2.f. The report shall contain an analysis and interpretation of the Type A test results and a summary analysis of periodic Type B and Type C tests that were performed since the last Type A test.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74  
License No. DPR-56

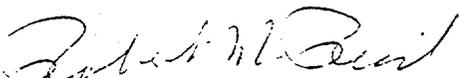
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 8, 1980, as supplemented October 3, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 74, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

iii

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257

Insert Pages

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vii (new page)

240n-240q (new pages)

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PBAPS

TABLE OF CONTENTS (cont'd)

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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.15 Seismic Monitoring  
InstrumentationApplicability

Applies to the operational status of the seismic monitoring instrumentation.

Specifications

- A. The seismic monitoring instrumentation shown in Table 3.15 shall be operable.
- B. With one or more seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.2, prepare and submit a Special Report to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within the next 10 working days outlining the cause of the malfunction and the plans for restoring the instrument(s) to operable status.
- C. The provisions of Specification 3.0.c are not applicable.

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Applies to the surveillance requirements of the seismic monitoring instrumentation.

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- A. Each of the required seismic monitoring instruments shall be demonstrated operable by the performance of the Instrument Check, Instrument Functional Test, and Instrument Calibration operations at the frequencies shown in Table 4.15.
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1. Triaxial Time-History Accelerographs		
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b. Refueling Floor	0.1-10g	1
c. RCIC Pump (Rm #7)	0.1-10g	1
d. "C" Diesel Generator	0.1-10g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01-2g	1
b. Refueling Floor	0.01-2g	1
c. "C" Diesel Generator	0.01-2g	1
3. Triaxial Response-Spectrum Recorders		
a. Cable Spreading Rm	0.1-10g	1*

\* With reactor control room annunciation

\*\* Effective upon completion of installation.

# Seismic instrumentation located in Unit 2

TABLE 4.15\*\*

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instruments and Sensor Locations#</u>	<u>Instrument* Check</u>	<u>Instrument* Functional Test</u>	<u>Instrument* Calibration</u>
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<b>2. Triaxial Peak Accelerographs</b>			
a. Reactor Piping (Drywell)	NA	NA	R
b. Refueling Floor	NA	NA	R
c. "C" Diesel Generator	NA	NA	R
<b>3. Triaxial Response-Spectrum Recorders</b>			
a. Cable Spreading Rm	M	SA	R

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M: every month  
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## 6.9.2 Continued

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
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- b. Reactor vessel inservice inspection, Specification 3.6.G and 4.6.G within 90 days of the completion of the reviews.
- c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20535

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMENDMENT NO. 75 TO LICENSE NO. DPR-44  
AND  
AMENDMENT NO. 74 TO LICENSE NO. DPR-56

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

PHILADELPHIA ELECTRIC COMPANY

DOCKETS NOS. 50-277 AND 50-278

I. INTRODUCTION

By letter dated August 8, 1980, Philadelphia Electric Company (licensee) proposed amendments to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. The amendments would revise the Technical Specifications by adding operability and surveillance requirements for seismic monitoring instrumentation. The licensee's application is in response to our request dated July 1, 1980.

II. EVALUATION

By our letter dated July 1, 1980, we indicated to the licensee that a review of the Technical Specifications for Peach Bottom 2 and 3 reflected that they did not include Limiting Conditions for Operation or Surveillance Requirements for seismic instrumentation. We requested that the licensee propose appropriate Technical Specifications so that verification of his conformance with the requirements of Appendix A to 10 CFR Part 100 could be assured. In response to our request the licensee indicated that a new seismic monitoring system which would comply with the guidelines of Regulatory Guide 1.12 was initiated in the spring of 1980; delivery was expected in September 1980; and the equipment would be operational by the fall of 1980. The system to be installed consists of four (4) triaxial time-history accelerographs, three (3) triaxial peak accelerographs, and a triaxial response-spectrum recorder. On September 24, 1980, we discussed, via telephone, the proposed system. The licensee stated that the response-spectrum recorder located in the cable spreading room has two sets of sensors associated with it: one triaxial time-history and one response-spectrum monitor, both located on the containment foundation. This information was documented in the licensee's letter dated October 3, 1980. Based on this information, we have determined that the proposed system is in conformance with our position set forth in Regulatory Guide 1.12 and is therefore acceptable. We suggested to the licensee that the triaxial response-spectrum recorder description be included in the bases to clearly demonstrate consistency with Regulatory Guide 1.12 regulatory position. He agreed.

8012010081

The licensee's submittal of Limiting Conditions for Operation for this equipment and associated Surveillance Requirements is consistent with current licensing practice as set forth in NUREG-0123, "Standard Technical Specifications for General Electric Boiling Water Reactors". Accordingly, the licensee's submittal, as amended by the staff, is acceptable.

### III. ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

### IV. CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazard consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 19, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-277 AND 50-278PHILADELPHIA ELECTRIC COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 75 and 74 to Facility Operating Licenses Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The amendments to the Technical Specifications add operability and surveillance requirements for a new seismic monitoring instrumentation system which is being installed at the facility.

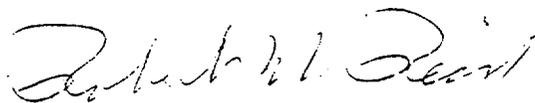
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated August 8, 1980, as supplemented October 3, 1980, (2) Amendments Nos. 75 and 74 to Licenses Nos. DPR-44 and DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of November 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing