

MAR 28 1983

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Mr. Edward G. Bauer, Jr.
 Vice President and General Counsel
 Philadelphia Electric Company
 2301 Market Street
 Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. DPR-56 for Peach Bottom Atomic Power Station, Unit No. 3. The amendment consists of changes to the Technical Specifications (TSS) in response to your formal application dated November 23, 1983. You originally requested this change by telephone on November 19, 1982, and confirmed the request by telecopied letter of the same date. Due to the urgent nature of your request, authorization was granted by telephone on November 19, 1982, to allow a one-time extension of the seven-day Limiting Condition for Operation to ten days in TS 3.5.A.5 regarding the operation of a residual heat removal heat exchanger.

Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

"ORIGINAL SIGNED BY:"

Gerald E. Gears, Project Manager
 Operating Reactors Branch #4
 Division of Licensing

Enclosures:

1. Amendment No. 91 to DPR-56
2. Safety Evaluation
3. Notice

cc w/enclosures:
 See next page

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No legal objection to approval of amendment or FR Notice. SER Review not requested.

OFFICE	ORB#4:DL RIngram	ORB#4:DL GGears CB	C-ORB#4:DL JStov	AD-OR:DL GLThas	OELD CUTCHIN		
SURNAME							
DATE	3/1/83	3/1/83	3/2/83	3/2/83	3/23/83		

Philadelphia Electric Company

cc w/enclosure(s):

Eugene J. Bradley
Philadelphia Electric Company
Assistant General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr.
1747 Pennsylvania Avenue, N.W.
Washington, D. C. 20006

Thomas A. Deming, Esq.
Assistant Attorney General
Department of Natural Resources
Annapolis, Maryland 21401

Philadelphia Electric Company
ATTN: Mr. W. T. Ullrich
Peach Bottom Atomic
Power Station
Delta, Pennsylvania 17314

Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Allen R. Blough
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Mr. Ronald C. Haynes, Regional Administrator
U. S. Nuclear Regulatory Commission, Region I
Office of Inspection and Enforcement
631 Park Avenue
King of Prussia, Pennsylvania 19406

Regional Radiation Representative
EPA Region III
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

M. J. Cooney, Superintendent
Generation Division - Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. DPR-56

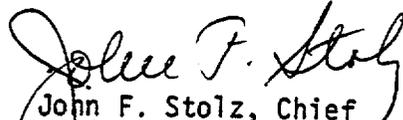
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) telecopied November 19, 1982, as confirmed November 23, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment became effective on November 19, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAR 28 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

126

Insert

126

LIMITING CONDITIONS FOR OPERATION

3.5.A Core Spray and LPCI Subsystem (cont'd)

3. Two independent Low Pressure Coolant Injection (LPCI) subsystems will be operable with each subsystem comprised of:
- a. (Two 33-1/3%) capacity pumps,
 - b. An operable flow path capable of taking suction from the suppression pool and transferring the water to the reactor pressure vessel, and
 - c. During power operation the LPCI system cross-tie valve closed and the associated valve motor operator circuit breaker locked in the off position.

Both LPCI subsystems shall be operable whenever irradiated fuel is in the reactor vessel, and prior to reactor startup from the Cold Shutdown Condition, except as specified in 3.5.A.4 and 3.5.A.5 below.

4. From and after the date that one of the four LPCI pumps is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days provided that during such seven days the remaining active components of the LPCI subsystems, and all active components of both core spray subsystems are operable.
- *5. From and after the date that one LPCI subsystem is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 7 days unless it is sooner made operable, provided that during such 7 days all active components of both core spray subsystems and the remaining LPCI subsystem are operable.

*This 7-day LCO is temporarily extended to 10 days, expiring November 24, 1982.

SURVEILLANCE REQUIREMENTS

4.5.A Core Spray and LPCI Subsystem (cont'd)

<u>Item</u>	<u>Frequency</u>
(c) Motor Operated valve operability	Once/month
(d) Pump Flow Rate	Once/3 months
Each LPCI pump shall deliver 10,900 gpm against a system head corresponding to a vessel pressure of 20 psig based on individual pump tests.	
(e) Operability check to ensure that pumps will start and motor operated injection valves will open	In accordance with 4.5.A.2, 4.5.A.4 and 4.5.A.5

4. When it is determined that one of the RHR (LPCI) pumps is inoperable at a time when it is required to be operable, the remaining LPCI pumps and associated flow paths and both core spray subsystems shall be demonstrated to be operable in accordance with 4.5.A.1(f) and 4.5.A.3(e) within 24 hours and at least once per 72 hours thereafter until the LPCI subsystem is restored to operable status.
5. When it is determined that one of the LPCI subsystem is inoperable both core spray subsystems and the remaining LPCI subsystem shall be demonstrated to be operable within 24 hours, and at least once per 72 hours thereafter until the LPCI subsystem is restored to operable status.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE NO. DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

DOCKET NO. 50-278

Introduction

By letter dated November 19, 1982, as confirmed by application dated November 23, 1982, the Philadelphia Electric Company (PECo or the licensee) made application to amend the Technical Specifications (TSs) for the Peach Bottom Atomic Power Station, Unit No. 3, to allow for a one-time extension of the seven-day Limiting Condition for Operation (LCO) to 10 days in TS 3.5.A.5 regarding the operability of a Residual Heat Removal (RHR) heat exchanger.

Evaluation

The low pressure coolant injection (LPCI) system at Peach Bottom Unit No. 3 is divided into two 100% capacity subsystems or "loops". On November 14, 1982, the licensee declared that one loop of the LPCI system was inoperable due to a tube leak in the RHR heat exchanger serving this loop. This action, as specified in TS 3.5.A.5, placed the loop in a seven day LCO mode. The secondary redundant loop, was determined to be operable. By telecon on November 19, 1982, the licensee requested a three-day extension of the seven-day LCO to November 24, 1982.

- We evaluated the ability of the plant to provide minimum engineered safety features (ESF) in the event of a Loss of Coolant Accident (LOCA), assuming one RHR heat exchanger inoperable, a loss of offsite power and a variety of additional single failures, such as loss of any diesel generators, an additional heat exchanger or pump. In addition, we evaluated the ability of the plant to safely shut down in the event of a loss of power to both Peach Bottom Units 2 and 3 with a single failure.

We concluded that a three-day extension of the seven-day LCO with one RHR heat exchanger, out of four, inoperable, and a LOCA occurring during the interval, with a concurrent loss of offsite power and an additional single failure, would not leave the plant with less than the minimum ESF needed in such an event. We further conclude that the plant, with an RHR heat exchanger inoperable, with an additional single failure and a loss of offsite power, would be able to provide sufficient decay heat removal, to both units, to attain a safe shutdown condition.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: MAR 28 1983

The following NRC personnel have contributed to this Safety Evaluation:
M. Fairtile, G. Holahan, J. T. Beard.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-278PHILADELPHIA ELECTRIC COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 91 to Facility Operating License No. DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications (TSs) for operation of the Peach Bottom Atomic Power Station, Unit No. 3 (the facility) located in York County, Pennsylvania.

The amendment was authorized by telephone on November 19, 1982. It revised TS 3.5.A.5 to allow a one-time extension of the seven-day Limiting Condition for Operation to 10 days regarding the operation of a Residual Heat Removal heat exchanger. The amendment was issued on an expedited basis to avoid an unscheduled plant shutdown.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

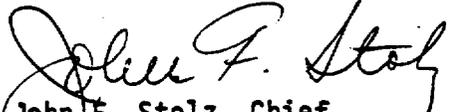
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant

to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 23, 1982, (2) the Commission's letter to Philadelphia Electric Company dated November 23, 1982, (3) Amendment No. 91 to License No. DPR-56, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of March 1983.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing