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Dockets Nos. 50-277  
and 50-278

Philadelphia Electric Company  
 ATTN: Mr. Edward G. Bauer, Jr., Esquire  
 Vice President and General Counsel  
 2301 Market Street  
 Philadelphia, Pennsylvania 19101

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 35 and 35 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications and are in response to your request dated March 8, 1977.

These amendments to the Technical Specifications will modify the method of testing the operability of relief valves at Units Nos. 2 and 3.

Copies of the Safety Evaluation and the FEDERAL REGISTER Notice are also enclosed.

Sincerely,

Original signed by

George Lear, Chief  
 Operating Reactors Branch #3  
 Division of Operating Reactors

Enclosures:

1. Amendments Nos. 35 and 35
2. Safety Evaluation
3. FEDERAL REGISTER Notice

cc: See page 2

*Constructive*

*[Signature]*

OFFICE >	ORB #3	ORB #3	OELD	ORB #3		
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Philadelphia Electric Company

- 2 -

cc:

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Philadelphia, Pennsylvania 19101

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Philadelphia Electric Company  
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Albert R. Steel, Chairman  
Board of Supervisors  
Peach Bottom Township  
R. D. #1  
Delta, Pennsylvania 17314



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35  
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, (the licensees) dated March 8, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

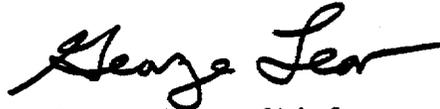
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 30, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 35

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages, 148 and 157, are also provided to maintain document completeness. No changes were made to pages 148 and 157.

Remove

147  
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Replace

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PRAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.6.D Safety and Relief Valves

1. During reactor power operating conditions and prior to reactor startup from a Cold Condition, or whenever reactor coolant pressure is greater than atmospheric and temperature greater than 212<sup>o</sup>F, both safety valves and the safety modes of all relief valves shall be operable, except as specified in 3.6.D.2.
2.
  - (a) From and after the date that the safety valve function of one relief valve is made or found to be inoperable, continued reactor operation is permissible only during the succeeding thirty days unless such valve function is sooner made operable.
  - (b) From and after the date that the safety valve function of two relief valves is made or found to be inoperable, continued reactor operation is permissible only during the succeeding seven days unless such valve function is sooner made operable.
3. If Specification 3.6.D.1 is not met, an orderly shutdown shall be initiated and the reactor coolant pressure shall be reduced to atmospheric within 24 hours.

4.6.D Safety and Relief Valves

1. At least one safety valve and 5 relief valves shall be checked or replaced with bench checked valves once per operating cycle. All valves will be tested every two cycles.  

The set point of the safety valves shall be as specified in Specifications 2.2.
2. At least one of the relief valves shall be disassembled and inspected each refueling outage.
3. The integrity of the relief safety valve bellows shall be continuously monitored. The switches shall be calibrated once per operating cycle. The accumulators and air piping shall be inspected for leakage using leak test fluid once per operating cycle.
4. With the reactor pressure  $\geq$  100 psig, each relief valve shall be manually opened once per operating cycle. Verification that each relief valve has opened shall either be by observation of compensating turbine bypass valve closure or load reduction or change in measured steam flow depending on the operating configuration existing during the test.

LIMITING CONDITIONS FOR OPERATION3.6. E. Jet Pumps

1. Whenever the reactor is in the startup or run modes, all jet pumps shall be operable. If it is determined that a jet pump is inoperable, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown Condition within 24 hours.
2. Flow indications from each of the 20 jet pumps shall be verified prior to initiation of reactor startup from a cold shutdown condition.
3. The indicated core flow is the sum of the flow indication from each of the 20 jet pumps. If flow indication failure occurs for two or more jet pumps immediate corrective action shall be taken. If flow indication for all but 1 jet pump cannot be obtained within 12 hours an orderly shutdown shall be initiated, and the reactor shall be in a cold shutdown condition within 24 hours.

SURVEILLANCE REQUIREMENTS4.6.E Jet Pumps

1. Whenever there is recirculation flow with the reactor in the startup or run modes, jet pump operability shall be checked daily by verifying that the following conditions do not occur simultaneously:
  - (a) The two recirculation loops have a flow imbalance of 15% or more when the pumps are operated at the same speed.
  - (b) The indicated value of core flow rate varies from the value derived from loop flow measurements by more than 10%.
  - (c) The diffuser to lower plenum differential pressure reading on an individual jet pump varies from the mean of all jet pump differential pressures by more than 10%.
2. Additionally when operating with one recirculation pump with the equalizer valves closed, the diffuser to lower plenum differential pressure shall be checked daily and the differential pressure of any jet pump in the idle loop shall not vary by more than 10% from established pattern.
3. The baseline data required to evaluate the conditions in specification 4.6.E.1 and 4.6.E.2 will be obtained each operating cycle.

3.6.D & 4.6.D BASES

Safety and Relief Valves

The safety and relief valves are required to be operable above the pressure (122 psig) at which the core spray system is not designed to deliver full flow. The pressure relief system for each unit at the Peach Bottom APS has been sized to meet two design bases. First, the total safety/relief valve capacity has been established to meet the overpressure protection criteria of the ASME code. Second, the distribution of this required capacity between safety valves and relief valves has been set to meet design basis 4.4.4.1 of subsection 4.4 which states that the nuclear system relief valves shall prevent opening of the safety valves during normal plant isolations and load rejections.

The details of the analysis which shows compliance with the ASME code requirements is presented in subsection 4.4 of the FSAR and the Reactor Vessel Overpressure Protection Summary Technical Report presented in Appendix K of the FSAR.

Thirteen safety/relief valves have been installed on Peach Bottom Unit 2 with a total capacity of 78.2% of rated steam flow. The analysis of the worst overpressure transient, (3 second closure of all main steam line isolation valves) neglecting the direct scram (valve position scram) results in a maximum vessel pressure of 1285 psig if a neutron flux scram is assumed. This results in a 89 psig margin to the code allowable overpressure limit of 1375 psig.

To meet the power generation design basis, the total safety/relief capacity of 78.2% has been divided into 64.5% relief (11 valves) and 13.7% safety (2 valves). The analysis of the plant isolation transient (Turbine trip with bypass valve failure to open) assuming a turbine trip scram is presented in Section 6.3 of NEDO-21172, Revision 1 for Peach Bottom Unit 2. This analysis shows that the 11 relief valves limit pressure at the safety valves to 39 psig below the setting of the safety valves. Therefore, the safety valves will not open.

Experience in relief and safety valve operation shows that a testing of 50 per cent of the valves per year is adequate to detect failure or deteriorations. The relief and safety valves are benchtested every second

3.6.D & 4.6.D BASES (Cont'd.)

operating cycle to ensure that their set points are within the  $\pm 1$  percent tolerance. Additionally, once per operating cycle, each relief valve is tested manually with reactor pressure above 100 psig to demonstrate its ability to pass steam.

Observations of a change in the steam flow to the turbine either by observation of closure of the control valves with resultant load drop, or closure of the by-pass valves, or change in measured steam flow are all indications that a relief valve has opened diverting some of the reactor steam flow from the turbine to the torus.

The requirements established above apply when the nuclear system can be pressurized above ambient conditions. These requirements are applicable at nuclear system pressures below normal operating pressures because abnormal operational transients could possibly start at these conditions such that eventual overpressure relief would be needed. However, these transients are much less severe, in terms of pressure, than those starting at rated conditions. The valves need not be functional when the vessel head is removed, since the nuclear system cannot be pressurized.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35  
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, (the licensees) dated March 8, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

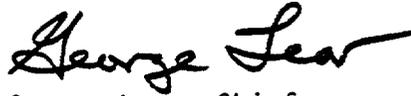
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 30, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 35

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages, 148 and 157, are also provided to maintain document completeness. No changes were made on pages 148 and 157.

Remove

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Replace

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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.6.D Safety and Relief Valves

1. During reactor power operating conditions and prior to reactor startup from a Cold Condition, or whenever reactor coolant pressure is greater than atmospheric and temperature greater than 212°F, both safety valves and the safety modes of all relief valves shall be operable, except as specified in 3.6.D.2.
2.
  - (a) From and after the date that the safety valve function of one relief valve is made or found to be inoperable, continued reactor operation is permissible only during the succeeding thirty days unless such valve function is sooner made operable.
  - (b) From and after the date that the safety valve function of two relief valves is made or found to be inoperable, continued reactor operation is permissible only during the succeeding seven days unless such valve function is sooner made operable.
3. If Specification 3.6.D.1 is not met, an orderly shutdown shall be initiated and the reactor coolant pressure shall be reduced to atmospheric within 24 hours.

4.6.D Safety and Relief Valves

1. At least one safety valve and 5 relief valves shall be checked or replaced with bench checked valves once per operating cycle. All valves will be tested every two cycles.  
  
The set point of the safety valves shall be as specified in Specifications 2.2.
2. At least one of the relief valves shall be disassembled and inspected each refueling outage.
3. The integrity of the relief safety valve bellows shall be continuously monitored. The switches shall be calibrated once per operating cycle. The accumulators and air piping shall be inspected for leakage using leak test fluid once per operating cycle.
4. With the reactor pressure  $\geq$  100 psig, each relief valve shall be manually opened once per operating cycle. Verification that each relief valve has opened shall either be by observation of compensating turbine bypass valve closure or load reduction or change in measured steam flow depending on the operating configuration existing during the test.

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.6. E. Jet Pumps

1. Whenever the reactor is in the startup or run modes, all jet pumps shall be operable. If it is determined that a jet pump is inoperable, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown Condition within 24 hours.
2. Flow indications from each of the 20 jet pumps shall be verified prior to initiation of reactor startup from a cold shutdown condition.
3. The indicated core flow is the sum of the flow indication from each of the 20 jet pumps. If flow indication failure occurs for two or more jet pumps immediate corrective action shall be taken. If flow indication for all but 1 jet pump cannot be obtained within 12 hours an orderly shutdown shall be initiated, and the reactor shall be in a cold shutdown condition within 24 hours.

4.6.E Jet Pumps

1. Whenever there is recirculation flow with the reactor in the startup or run modes, jet pump operability shall be checked daily by verifying that the following conditions do not occur simultaneously:
  - (a) The two recirculation loops have a flow imbalance of 15% or more when the pumps are operated at the same speed.
  - (b) The indicated value of core flow rate varies from the value derived from loop flow measurements by more than 10%.
  - (c) The diffuser to lower plenum differential pressure reading on an individual jet pump varies from the mean of all jet pump differential pressures by more than 10%.
2. Additionally when operating with one recirculation pump with the equalizer valves closed, the diffuser to lower plenum differential pressure shall be checked daily and the differential pressure of any jet pump in the idle loop shall not vary by more than 10% from established pattern.
3. The baseline data required to evaluate the conditions in specification 4.6.E.1 and 4.6.E.2 will be obtained each operating cycle.

### 3.6.D & 4.6.D BASES

#### Safety and Relief Valves

The safety and relief valves are required to be operable above the pressure (122 psig) at which the core spray system is not designed to deliver full flow. The pressure relief system for each unit at the Peach Bottom APS has been sized to meet two design bases. First, the total safety/relief valve capacity has been established to meet the overpressure protection criteria of the ASME code. Second, the distribution of this required capacity between safety valves and relief valves has been set to meet design basis 4.4.4.1 of subsection 4.4 which states that the nuclear system relief valves shall prevent opening of the safety valves during normal plant isolations and load rejections.

The details of the analysis which shows compliance with the ASME code requirements is presented in subsection 4.4 of the FSAR and the Reactor Vessel Overpressure Protection Summary Technical Report presented in Appendix K of the FSAR.

Thirteen safety/relief valves have been installed on each unit with a total capacity of 74% of design steam flow. The analysis of the worst overpressure transient, (3-second closure of all main steam line isolation valves) neglecting the direct scram (valve position scram) results in a maximum vessel pressure of 1203 psig if a pressure scram is assumed, or 1250 psig if a neutron flux scram is assumed. This results in 73 and 115 psig margins respectively to the code allowable overpressure limit of 1375 psig. In addition, the same event was analyzed to determine the number of installed valves which must open to limit peak pressure to 1350 psig (25 psig margin). The results of this analysis shows that seven valves must open if a neutron flux scram is assumed or ten valves must open if a pressure scram is assumed.

To meet the power generation design basis, the total safety/relief capacity of 74% has been divided into 61% relief (11 valves) and 13% safety (2 valves). The analysis of the plant isolation transient (Turbine trip with bypass valve failure to open) assuming a turbine trip scram is presented in FSAR paragraph 14.5.1a & b. This analysis shows that the 11 relief valves limit pressure at the safety valves to 1168 psig, well below the setting of the safety valves. Therefore, the safety valves will not open. This analysis shows that peak system pressure is limited to 1210 psig which is 165 psig below the allowed vessel overpressure of 1375 psig.

Experience in relief and safety valve operation shows that a testing of 50 per cent of the valves per year is adequate to detect failure or deteriorations. The relief and safety valves are benchtested every second

3.6.D & 4.6.D BASES (Cont'd.)

operating cycle to ensure that their set points are within the  $\pm$  1 percent tolerance. Additionally, once per operating cycle, each relief valve is tested manually with reactor pressure above 100 psig to demonstrate its ability to pass steam.

Observations of a change in the steam flow to the turbine either by observation of closure of the control valves with resultant load drop, or closure of the by-pass valves, or change in measured steam flow are all indications that a relief valve has opened diverting some of the reactor steam flow from the turbine to the torus.

The requirements established above apply when the nuclear system can be pressurized above ambient conditions. These requirements are applicable at nuclear system pressures below normal operating pressures because abnormal operational transients could possibly start at these conditions such that eventual overpressure relief would be needed. However, these transients are much less severe, in terms of pressure, than those starting at rated conditions. The valves need not be functional when the vessel head is removed, since the nuclear system cannot be pressurized.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NOS. 35 AND 35 TO FACILITY LICENSE NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY  
PEACH BOTTOM ATOMIC POWER STATION

UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 35 AND 35 TO FACILITY LICENSE NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION

UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

INTRODUCTION

By letter dated March 8, 1977, Philadelphia Electric Company (PECo) proposed amendments to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station Unit Nos. 2 and 3. The proposed changes would modify the method of testing the operability of relief valves at Unit Nos. 2 and 3.

EVALUATION

The staff has recently become aware of a potential deficiency in the method being used to confirm valve operability during periodic testing of Boiling Water Reactor (BWR) safety-relief valves. This deficiency concerns the use of the safety-relief valve temperature indication as a positive method of confirmation that a safety-relief valve is open when manually actuated during surveillance testing.

We have found that an increased temperature indication may be obtained at the safety-relief valve exit with the safety-relief valve closed. This indicated temperature increase is the result of steam vented through the valve actuation mechanism during the surveillance test. In view of this finding, we have concluded that a temperature increase at the valve exit, by itself, does not provide a positive means of verification that the safety-relief valve has opened.

When we became aware of the above deficiency in some BWR Technical Specifications, we requested PECo, by our letter dated January 3, 1977, to propose a change to the Technical Specifications of their boiling water reactors. We provided examples of acceptable methods that PECo could propose to provide assurance of satisfactory operation of the relief valves. In response to our request, PECo submitted their March 8, 1977 request for amendments in Licenses Nos. DPR-44 and DPR-56.

We have reviewed PECO's proposed change to the Technical Specifications for Peach Bottom Units Nos. 2 and 3. PECO has proposed the deletion of the specification based on relief valve exit temperature and replaced it with a specification based upon the observation of the turbine bypass valve closure load reduction, or change in measured steam flow that occurs concurrent with the actual opening of the relief valve. Closure of the turbine valves or the control valves compensates for the diversion of steam to the suppression pool.

We have concluded that the method proposed by PECO provides assurance that the relief valve has actually opened during surveillance testing and is therefore acceptable to the NRC staff.

#### ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 30, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 35 and 35 to Facility Operating Licenses Nos. DPR-44 and DPR-56, respectively, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3, located in Peach Bottom, York County, Pennsylvania. The amendments are effective as of the date of issuance.

These amendments to the Technical Specifications will modify the method of testing the operability of relief valves at Units Nos. 2 and 3.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

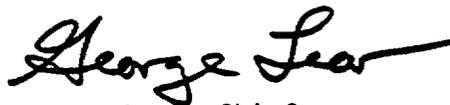
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant

to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated March 8, 1977, (2) Amendments Nos. 35 and 35 to Licenses Nos. DPR-44 and DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Martin Memorial Library, 159 E. Market Street, York, Pennsylvania 17401. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 30th day of June 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors